

October 9, 1985

Docket No. 50-293

Mr. William D. Harrington
Senior Vice President, Nuclear
Boston Edison Company
800 Boylston Street
Boston, Massachusetts 02199

Dear Mr. Harrington:

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated June 18, 1985.

The amendment changes the Technical Specifications by changing the Reactor Low Water Level (inside shroud) trip requirement to " ≥ 307 inches above vessel zero (approximately 2/3 core height)."

A copy of the related Safety Evaluation is also enclosed.

Sincerely,
Original signed by/

Paul H. Leech, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 90 to License No. DPR-35
2. Safety Evaluation

cc w/enclosures:
See next page

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MVirgilio

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GLainas
09/18/85

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Mr. William D. Harrington
Boston Edison Company
Pilgrim Nuclear Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Boston Edison Company (the licensee) dated June 18, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

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B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 9, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following page of the Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the areas of change.

Remove

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Insert

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TABLE 3.2.B
INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

<u>Minimum # of Operable Instrument Channels Per Trip System (1)</u>	<u>Trip Function</u>	<u>Trip Level Setting</u>	<u>Remarks</u>
2	Reactor Low-Low Water Level	at or above -49 in. indicated level (4)	<ol style="list-style-type: none"> 1. In conjunction with Low Reactor Pressure, initiates Core Spray and LPCI. 2. In conjunction with High Drywell Pressure, 120 second time delay and LPCI or Core Spray pump interlock initiates Auto Blowdown (ADS). 3. Initiates HPCI; RCIC. 4. Initiates starting of Diesel Generators.
2	Reactor High Water Level	<+48" indicated level	Trips HPCI and RCIC turbines.
1	Reactor Low Water Level (inside shroud)	>307" above vessel zero (approximately 2/3 core height)	Prevents inadvertent operation of containment spray during accident condition.
2	Containment High Pressure	1 < p < 2 psig	Prevents inadvertent operation of containment spray during accident condition.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-35
BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

1.0 INTRODUCTION

By letter dated June 18, 1985 the Boston Edison Company (licensee) proposed a change in Table 3.2.B relative to the trip level setting for Reactor Low Water Level (inside shroud). The change consists of replacing the setting specification of " ≥ 302 inches above vessel zero (2/3 core height)" with " ≥ 307 inches above vessel zero (approximately 2/3 core height)."

2.0 EVALUATION

The proposed change would raise the minimum trip level setting for Reactor Low Water Level by 5 inches to 307 inches above vessel zero. This is a correction to make the setting more nearly equivalent to the general term "2/3 core height". The 307-inch level above vessel zero will serve the purpose of the minimum setpoint, which is to prevent inadvertent operation of the containment spray during an accident condition. The 5-inch higher minimum level will also provide a somewhat greater degree of core coverage during an accident condition. On this basis, we find that safety considerations would not be adversely affected by the proposed change. It is, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

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4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. H. Leech

Dated: October 9, 1985