February 22, 1995

Mr. Ted C. Feigenbaum
Senior Vice President
and Chief Nuclear Officer
North Atlantic Energy Service Corporation
Post Office Box 300
Seabrook, NH 03874

SUBJECT: CORRECTION TO AMENDMENT 34 TO FACILITY OPERATING LICENSE NPF-86 (TAC M86712)

Dear Mr. Feigenbaum:

On January 26, 1995, the Commission issued Amendment 34 to Facility Operating License NPF-86 for the Seabrook Station, Unit No. 1. It has been brought to my attention that revised Basis page B 3/4 2-4 inadvertently was not included. Please insert the enclosed replacement page B 3/4 2-4. For convenience, overleaf page B 3/4 2-3 is included. A corrected Attachment to License Amendment 34 is also included.

Sincerely,

Original signed by:

Albert W. De Agazio, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-443
Serial No. SEA-95-004

Enclosure: 1. Replacement page B 3/4 2-4 to Amendment 34
2. Corrected Attachment to Amendment 34

cc w/encl: See next page

DISTRIBUTION:

Docket File	PMcKee	CGrimes
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OFFICE	LA:PDI-4	PM:PDI-4	D:PDI-4		
NAME	SNorris	ADeAgazio:bf	PMcKee		
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf pages have been provided.*

Remove	Insert
ix	ix
x*	x*
xiii*	xiii*
xiv	xiv
xv	xv
1-5	1-5
1-6*	1-6*
B 2-7	B 2-7
B 2-8*	B 2-8*
3/4 3-13	3/4 3-13
3/4 3-14*	3/4 3-14*
3/4 3-49	3/4 3-49
3/4 3-50*	3/4 3-50*
3/4 4-1*	3/4 4-1*
3/4 4-2	3/4 4-2
3/4 6-17	3/4 6-17
3/4 6-18*	3/4 6-18*
3/4 7-3	3/4 7-3
3/4 7-4*	3/4 7-4*

Remove	Insert
3/4 10-5*	3/4 10-5*
3/4 10-6	3/4 10-6
B 3/4 1-3	B 3/4 1-3
B 3/4 1-4*	B 3/4 1-4*
B 3/4 2-3*	B 3/4 2-3*
B 3/4 2-4	B 3/4 2-4
B 3/4 3-1*	B 3/4 3-1*
B 3/4 3-2	B 3/4 3-2
B 3/4 4-5	B 3/4 4-5
B 3/4 4-6*	B 3/4 4-6*
B 3/4 10-1	B 3/4 10-1
5-9	5-9
5-10*	5-10*
6-5	6-5
6-6	6-6
6-7	6-7
6-8	6-8
---	6-8A
---	6-8B
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-13	6-13

Remove	Insert
6-14	6-14
---	6-14A
6-17*	6-17*
6-18	6-18

3/4.2.2 and 3/4.2.3 HEAT FLUX HOT CHANNEL FACTOR and NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR (Continued)

$F^N_{\Delta H}$ will be maintained within its limits provided Conditions a. through d. above are maintained. The design limit DNBR includes margin to offset any rod bow penalty. Margin is also maintained between the safety analysis limit DNBR and the design limit DNBR. This margin is available for plant design flexibility.

When an F_Q measurement is taken, an allowance for both measurement error and manufacturing tolerance must be made. An allowance of 5% is appropriate for a full-core map taken with the movable incore detectors, while 5.21% is appropriate for surveillance results determined with the fixed incore detectors. A 3% allowance is appropriate for manufacturing tolerance.

For operation with the Fixed Incore Detector System (FIDS) Alarm OPERABLE, the cycle-dependent normalized axial peaking factor, K(Z), specified in COLR accounts for axial power shape sensitivity in the LOCA analysis. Assurance that the $F_Q(Z)$ limit on Specification 3.2.2 is met during both normal operation and in the event of xenon redistribution following power changes is provided by the FIDS Alarm through the plant process computer. This assures that the consequences of a LOCA would be within specified acceptance criteria.

For operation with the FIDS Alarm inoperable, the cycle-dependent normalized axial peaking factor, K(Z), specified in COLR accounts for possible xenon redistribution following power changes in addition to axial power shape sensitivity in the LOCA analysis. This assures that the consequences of a LOCA would be within specified acceptance criteria.

When RCS $F^N_{\Delta H}$ is measured, no additional allowances are necessary prior to comparison with the established limit. A bounding measurement error of 4.13% for $F^N_{\Delta H}$ has been allowed for in determination of the design DNBR value.

3/4.2.4 QUADRANT POWER TILT RATIO

The purpose of this specification is to detect gross changes in core power distribution between monthly Incore Detector System surveillances. During normal operation the QUADRANT POWER TILT RATIO is set equal to zero once acceptability of core peaking factors has been established by review of incore surveillances. The limit of 1.02 is established as an indication that the power distribution has changed enough to warrant further investigation.

3/4.2.5 DNB PARAMETERS

The limits on the DNB-related parameters assure that each of the parameters is maintained within the normal steady-state envelope of operation assumed in the transient and accident analyses. The limits are consistent with the initial FSAR assumptions and have been analytically demonstrated adequate to maintain a minimum DNBR of 1.30 throughout each analyzed transient. Operating procedures include allowances for measurement and indication uncertainty so that the limits of 594.3°F for T_{avg} and 2205 psig for pressurizer pressure are not exceeded.

The measurement error of 2.4% for RCS total flow rate is based upon performing a precision heat balance and using the result to normalize the RCS flow rate indicators. Potential fouling of the feedwater venturi which might not be detected could bias the result from the precision heat balance in a nonconservative manner. Therefore, a penalty of 0.1% for undetected fouling of the feedwater venturi is applied. Any fouling which might bias the RCS flow rate measurement greater than 0.1% can be detected by monitoring and trending various plant performance parameters. If detected, action shall be taken before performing subsequent precision heat balance measurements, i.e., either the effect of the fouling shall be quantified and compensated for in the RCS flow rate measurement or the venturi shall be cleaned to eliminate the fouling.

The 12-hour periodic surveillance of these parameters through instrument readout is sufficient to ensure that the parameters are restored within their limits following load changes and other expected transient operation.

The periodic surveillance of indicated RCS flow is sufficient to detect only flow degradation which could lead to operation outside the specified limit.