

October 21, 1982
Docket No. 50-293

Mr. A. Victor Morisi, Manager
Nuclear Operations Support Department
Boston Edison Company
25 Braintree Hill Park
Rockdale Street
Braintree, Massachusetts 02184

Dear Mr. Morisi:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated September 30, 1982.

The TS changes involve revisions to clarify conditions for which containment isolation valve operability is required and actions to be taken in the event of containment isolation valve inoperability.

The proposed change to Specification 3.7.D.1 "Primary Containment Isolation Valves" modifies the TSs to require containment isolation valve operability whenever primary containment integrity is required vice during reactor power operating conditions. This change makes Specification 3.7.D.1 consistent with Specification 3.7.A.2, which requires primary containment integrity whenever the reactor water temperature is greater than 212°F (with fuel in the reactor vessel). Primary containment integrity necessitates containment isolation valve operability or deactivation of inoperable valves in the isolated position.

The second change proposed by Boston Edison would change the wording of Specification 3.7.D.2 to more clearly define the intent of this specification and the category of valve (i.e., containment isolation valve) which may be used to satisfy the "Action" statement of this specification. The clarification of the intent of Specification 3.7.D.2 is accomplished through deletion of the words "reactor power operation may continue" from 3.7.D.2, thereby eliminating possible confusion as to the permissibility of initiating power operation with the requirements of Specification 3.7.D.2 otherwise satisfied.

We have reviewed these proposed changes and have determined that the changes would clarify the intent of and make the Pilgrim TSs more internally consistent. Consequently, we find them acceptable.

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We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No.63 to DPR-35
- 2. Notice

cc w/enclosures:
See next page

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Mr. A. Victor Morisi
Boston Edison Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Boston Edison Company (the licensee) dated September 30, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment: .
Changes to the Technical
Specifications

Date of Issuance: October 21, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace page 160 of the Appendix "A" Technical Specifications with the enclosed page 160.

3.7.0

Primary Containment Isolation Valves

1. Whenever primary containment integrity is required, all isolation valves listed in Table 3.7.1 and all instrument line flow check valves shall be operable except as specified in 3.7.D.2.
2. In the event any isolation valve specified in Table 3.7.1 becomes inoperable, at least one containment isolation valve in each line having an inoperable valve shall be placed in the isolated condition.
3. If Specification 3.7.D.1 and 3.7.D.2 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown condition within 24 hours.

4.7.D Primary Containment Isolation Valves

1. The primary containment isolation valves surveillance shall be performed as follows:
 - a. At least once per operating cycle the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and closure times.
 - b. At least once per quarter:
 - (1) All normally open power operated isolation valves (except for the main steam line power-operated isolation valves) shall be fully closed and reopened.
 - (2) Trip main steam isolation valves individually and verify closure time.
 - c. At least twice per week the main steam-line power-operated isolation valves shall be exercised by partial closure and subsequent reopening.
 - d. At least once per operating cycle the operability of the reactor coolant system instrument line flow check valves shall be verified.
2. Whenever an isolation valve listed in table 3.7.1 is inoperable, the position of at least one other valve in each line having an inoperable valve shall be recorded daily.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-293BOSTON EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 63 to Facility Operating License No. DPR-35 issued to Boston Edison Company (the licensee) which revised the Technical Specifications for operation of the Pilgrim Nuclear Power Station (the facility) located near Plymouth, Massachusetts. The amendment is effective as of the date of issuance.

The amendment modifies the Technical Specifications to clarify conditions for which containment isolation valve operability is required and actions to be taken in the event of containment isolation valve inoperability.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since it does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated September 30, 1982, (2) Amendment No. 63 to License No. DPR-35, and (3) the Commission's letter to the licensee dated October 21, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Plymouth Public Library on North Street in Plymouth, Massachusetts 02360. A single copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 21st day of October, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing