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Docket No. 50-293

APR 12 1982

Mr. A. Victor Morisi, Manager  
Nuclear Operations Support Department  
Boston Edison Company  
M/C Nuclear  
800 Boylston Street  
Boston, Massachusetts 02199



Dear Mr. Morisi:

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment consists of changes to the Technical Specifications in response to your application dated October 15, 1981.

These changes to the Technical Specifications pertain to inservice surveillance and operability requirements of hydraulic and mechanical snubbers in response to our letter of November 20, 1980 relating to snubber surveillance.

In reviewing your application, the BWR Standard Technical Specifications, NUREG-0123, Revision 3, served as the basis in assessing the acceptability of the proposed changes. The Standard Technical Specifications, pages 3/4 7-12 through 3/4 7-18, pertaining to snubbers (and the associated bases) are recognized by the staff as an acceptable implementation of the inservice surveillance and operability requirements pertaining to snubbers.

We have reviewed your submittal and find the proposed Technical Specification changes to be consistent with the BWR Standard Technical Specifications and therefore, conclude that these changes are acceptable.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

The amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or conse-

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Mr. A. V. Morisi

-2-

quences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 60 to DPR-35
2. Notice

cc w/enclosures:

See next page

OFFICE	ORB#2:DL	ORB#2:DL <i>KTB</i>	ORAB:DL	C-ORB#2:DL	AD-OR:DL	OELD	
SURNAME	SNorris	KEccleston	HShaw <i>H</i>	DVassallo	TRovak	<i>EC</i>	
DATE	3/3/82	3/31/82:cb	3/31/82	3/31/82	4/5/82	4/9/82	

Mr. A. Victor Morisi  
Boston Edison Company

cc:

Mr. Richard D. Machon  
Pilgrim Station Manager  
Boston Edison Company  
RFD #1, Rocky Hill Road  
Plymouth, Massachusetts 02360

Resident Inspector  
c/o U.S. NRC  
P.O. Box 867  
Plymouth, Massachusetts 02360

Henry Herrmann, Esquire  
Massachusetts Wildlife Federation  
151 Tremont Street  
Boston, Massachusetts 02111

Plymouth Public Library  
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Plymouth, Massachusetts 02360

Massachusetts Department of Public Health  
ATTN: Commissioner of Public Health  
600 Washington Street  
Boston, Massachusetts 02111

Water Quality & Environmental Commissioner  
Department of Environmental Quality  
Engineering  
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Ms. JoAnn Shotwell  
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Environmental Protection Division  
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U. S. Environmental Protection  
Agency  
Region I Office  
Regional Radiation Representative  
JFK Federal Building  
Boston, Massachusetts 02203

Ronald C. Haynes  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60  
License No. DPR-35

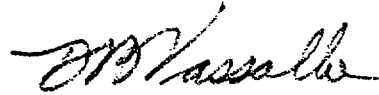
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Boston Edison Company (the licensee) dated October 15, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "D. B. Vassallo", written in a cursive style.

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment Number and contain a vertical line indicating the area of change.

Remove

137a  
137b  
137c  
137d  
137e  
-  
-  
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151a  
224

Replace

137a  
137b  
137c  
137d  
137e  
137f  
137g  
137h  
137i  
151a  
224

# LIMITING CONDITION FOR OPERATION

## 3.6.I Shock Suppressors (Snubbers)

1. During all modes of operation except Cold Shutdown and Refuel, all safety-related snubbers listed in Table 3.6.I(a) and 3.6.I(b) shall be operable except as noted in 3.6.I.2 through 3.6.I.3 below.

An Inoperable Snubber is a properly fabricated, installed and sized snubber which cannot pass its functional test.

Upon determination that a snubber is either improperly fabricated, installed or sized, the corrective action will be as specified for an inoperable snubber in Section 3.6.I.2.

2. From and after the time that a snubber is determined to be inoperable, replace or repair the snubber during the next 72 hours, and initiate an engineering evaluation to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubbers and to ensure that the supported component remains capable of meeting its intended function in the specific safety system involved.

Further corrective action for this snubber, and all generically susceptible snubbers, shall be determined by an engineering evaluation

3. From and after the time a snubber is determined to be inoperable, improperly fabricated, improperly installed or improperly sized, if the requirements of Section(s) 3.6.I.1 and 3.6.I.2 cannot be met, then the affected safety system, or affected portions of that system, shall be declared inoperable, and the limiting condition for that system entered, as appropriate.

# SURVEILLANCE REQUIREMENT

## 4.6.I Shock Suppressors (Snubbers)

The following surveillance requirements apply to all hydraulic and mechanical snubbers listed in Tables 3.6.I(a) and 3.6.I(b).

The required visual inspection interval varies inversely with the observed cumulative number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original time interval has elapsed may not be used to lengthen the required interval.

Number of snubbers found inoperable during inspection or during inspection interval:

<u>Inoperable Snubbers</u>	<u>Subsequent Visual Inspection Interval</u>
0	18 Months $\pm$ 25%
1	12 Months $\pm$ 25%
2	6 Months $\pm$ 25%
3,4	124 Days $\pm$ 25%
5,6,7	62 Days $\pm$ 25%
8 or more	31 Days

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

1. Visual Inspection Acceptance Criteria

A. Visual inspections shall verify:

## LIMITING CONDITION FOR OPERATION

4. Snubbers may be added to, or removed from, per 10 CFR 50.59, safety related systems without prior License Amendment to Table 3.6.I(a) or 3.6.I(b) provided that a revision to the appropriate Table is included with the next license amendment request.

## SURVEILLANCE REQUIREMENT

4.6.I Shock Suppressors (Snubbers)

1. That there are no visible indications of damage or impaired operability.
2. Attachments to the foundation or support structure are such that the functional capability of the snubber is not suspect.

B. Snubbers which appear INOPERABLE as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval provided that:

1. The cause of the rejection is clearly established and remedied for that particular snubber, and
2. The affected snubber is functionally tested, when necessary, in the as found condition and determined OPERABLE per specifications 4.6.I.2.B., 4.6.I.2.C., as applicable.
3. For any snubber determined inoperable per specification 4.6.I.2, clearly establish the cause of rejection and remedy the problem for that snubber, and any generically susceptible snubber.

2. Functional Tests (Hydraulic and Mechanical Snubbers)

A. Schedule

At least once per operating cycle (18 months), a representative sample (10% of the total of each type: hydraulic, mechanical) of snubbers in use in the plant shall be functionally tested, either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria of



# LIMITING CONDITION FOR OPERATION

# SURVEILLANCE REQUIREMENT

## 4.6.I Shock Suppressors (Snubbers)

Specification 4.6.I.2.B, or 4.6.I.2.C, as applicable, an additional 10% of that type of snubber shall be functionally tested.

### B. General Snubber Functional Test Acceptance Criteria (Hydraulic and Mechanical)

The general snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
2. Snubber release, or bleed-rate, as applicable, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

### C. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force. Drag force shall not have increased more than 50% since the last functional test.

### 3. Snubber Service Life Monitoring

#### A. A record of the service life

## LIMITING CONDITION FOR OPERATION

## SURVEILLANCE REQUIREMENT

4.6.I Shock Suppressors (Snubbers)

of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained.

- B. At least once per cycle, the installation and maintenance records for each snubber listed in Tables 3.6.I(a) and 3.6.I(b) shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated, or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.
- C. This Snubber Service Life Monitoring Program shall become effective July 1, 1982.

Table 3.6.I(a)

## SAFETY RELATED HYDRAULIC SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-1-10-1	Main Steam Line	42'			X (Drywell)	
SS-1-10-2	Main Steam Line	42'			X (Drywell)	
SS-1-10-3	Main Steam Line	42'			X (Drywell)	
SS-1-10-4	Main Steam Line	42'			X (Drywell)	
SS-1-10-5	Main Steam Line	42'			X (Drywell)	
SS-1-10-6	Main Steam Line	42'			X (Drywell)	
SS-1-10-7	Main Steam Line	42'			X (Drywell)	
SS-1-10-8	Main Steam Line	42'			X (Drywell)	
SS-1-10-9	Main Steam Line	42'			X (Drywell)	
SS-1-10-10	Main Steam Line	42'			X (Drywell)	
SS-1-10-11	Main Steam Line	42'			X (Drywell)	
SS-1-10-12	Main Steam Line	42'			X (Drywell)	
SS-6-10-6	Feedwater Sys.	41'			X (Drywell)	
SS-6-10-7	Feedwater Sys.	41'			X (Drywell)	
SS-6-10-8	Feedwater Sys.	44'			X (Drywell)	
SS-6-10-9	Feedwater Sys.	41'			X (Drywell)	
SS-6-10-10	Feedwater Sys.	44'			X (Drywell)	
SS-10-30-1	RHR System	52'			X (Drywell)	
SS-10-20-2	RHR System	52'			X (Drywell)	
SS-10-20-3	RHR System	52'			X (Drywell)	
SS-10-20-4	RHR System	52'			X (Drywell)	
SS-10-30-5	RHR System	24'			X (Drywell)	
SS-10-30-6	RHR System	24'			X (Drywell)	
SS-10-20-7	RHR System	24'			X (Drywell)	
SS-10-20-8	RHR System	24'			X (Drywell)	
SS-10-3-9	RHR System	87'			X (Drywell)	
SS-10-3-10	RHR System	90'			X (Drywell)	
SS-2-20-1	Recir. System	42'	X		X (Drywell)	
SS-2-20-2	Recir. System	42'	X		X (Drywell)	
SS-2-20-3	Recir. System	42'	X		X (Drywell)	
SS-2-20-4	Recir. System	42'	X		X (Drywell)	
SS-2-30-5	Recir. System	15'	X		X (Drywell)	
SS-2-30-6	Recir. System	15'	X		X (Drywell)	
SS-2-30-7	Recir. System	15'	X		X (Drywell)	

Table 3.6.I(a)

## SAFETY RELATED HYDRAULIC SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-2-30-8	Recir. System	15'	X		X (Drywell)	
SS-2-30-9	Recir. System	11'	X		X (Drywell)	
SS-2-30-10	Recir. System	11'	X		X (Drywell)	
SS-2-30-11	Recir. System	27'	X		X (Drywell)	
SS-2-30-12	Recir. System	27'	X		X (Drywell)	
SS-2-30-13	Recir. System	27'	X		X (Drywell)	
SS-2-30-14	Recir. System	27'	X		X (Drywell)	
SS-2-30-15	Recir. System	27'	X		X (Drywell)	
SS-2-30-16	Recir. System	27'	X		X (Drywell)	
SS-2-20-19	Recir. System	16'	X		X (Drywell)	
SS-2-20-20	Recir. System	16'	X		X (Drywell)	
SS-2-20-21	Recir. System	19'	X		X (Drywell)	
SS-2-20-22	Recir. System	16'	X		X (Drywell)	
SS-2-50-23	Recir. System	17'	X		X (Drywell)	
SS-2-20-24	Recir. System	18'	X		X (Drywell)	
SS-2-20-25	Recir. System	16'	X		X (Drywell)	
SS-2-50-26	Recir. System	16'	X		X (Drywell)	

Table 3.6.I(a)

## SAFETY RELATED HYDRAULIC SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-6-10-1	Feedwater System	42'			X (Drywell)	
SS-6-10-2	Feedwater System	42'			X (Drywell)	
SS-6-10-3	Feedwater System	42'			X (Drywell)	
SS-6-10-4	Feedwater System	42'			X (Drywell)	
SS-6-10-5	Feedwater System	42'			X (Drywell)	
SS-13-3-1	RCIC	38'			X (Drywell)	
SS-13-3-2	RCIC	38'			X (Drywell)	
SS-14-3-1	Core Spray	65'			X (Drywell)	
SS-14-3-2	Core Spray	65'			X (Drywell)	
SS-14-3-3	Core Spray	65'			X (Drywell)	
SS-14-3-4	Core Spray	65'			X (Drywell)	
SS-23-10-1	H.P.C.I.	42'			X (Drywell)	
SS-23-10-2	H.P.C.I.	42'			X (Drywell)	
SS-23-3-30	H.P.C.I.	-3' 09"			X	X H.P.C.I. Quadrant
SS-23-3-31	H.P.C.I.	-3' 09"			X	X H.P.C.I. Quadrant
SS-23-10-32	H.P.C.I.	-3' 09"			X	X H.P.C.I. Quadrant
SS-23-10-34	H.P.C.I.	-6'				X H.P.C.I. Quadrant
SS-23-10-35	H.P.C.I.	-6'				X H.P.C.I. Quadrant
SS-23-3-36	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-23-3-37	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-10-3-43	RHR	-3' 06"				X RHR Pump Room
SS-10-20-44	RHR	-3' 06"				X RHR Pump Room
SS-30-3-45	RBCCW	83' 5"				X Reactor Building
SS-10-10-46	RHR	6"				X Torus Compartment
H-3-1-1292	CRD System					

Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

Table 3.6.I(b)

## SAFETY RELATED MECHANICAL SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
MS-S-500	M.S. Relief Line A				X (Drywell)	
MS-S-501	M.S. Relief Line A				X (Drywell)	
MS-S-502	M.S. Relief Line A				X (Drywell)	
MS-S-503	M.S. Relief Line A				X (Drywell)	
MS-S-504	M.S. Relief Line A				X (Drywell)	
MS-S-505	M.S. Relief Line A				X (Drywell)	
MS-S-506	M.S. Relief Line A				X (Drywell)	
MS-S-507	M.S. Relief Line A				X (Drywell)	
MS-S-508	M.S. Relief Line B				X (Drywell)	
MS-S-509	M.S. Relief Line B				X (Drywell)	
MS-S-510	M.S. Relief Line B				X (Drywell)	
MS-S-511	M.S. Relief Line B				X (Drywell)	
MS-S-512	M.S. Relief Line B				X (Drywell)	
MS-S-513	M.S. Relief Line B				X (Drywell)	
MS-S-514	M.S. Relief Line B				X (Drywell)	
MS-S-515	M.S. Relief Line B				X (Drywell)	
MS-S-516	M.S. Relief Line C				X (Drywell)	
MS-S-517	M.S. Relief Line C				X (Drywell)	
MS-S-518	M.S. Relief Line C				X (Drywell)	
MS-S-519	M.S. Relief Line C				X (Drywell)	
MS-S-520	M.S. Relief Line C				X (Drywell)	
MS-S-521	M.S. Relief Line C				X (Drywell)	
MS-S-522	M.S. Relief Line C				X (Drywell)	
MS-S-523	M.S. Relief Line C				X (Drywell)	
MS-S-524	M.S. Relief Line C				X (Drywell)	

Table 3.6.I(b)

## SAFETY RELATED MECHANICAL SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
MS-S-525	M.S. Relief Line D				X (Drywell)	
MS-S-526	M.S. Relief Line D				X (Drywell)	
MS-S-527	M.S. Relief Line D				X (Drywell)	
MS-S-528	M.S. Relief Line D				X (Drywell)	
MS-S-529	M.S. Relief Line D				X (Drywell)	
MS-S-530	M.S. Relief Line D				X (Drywell)	
MS-S-531	M.S. Relief Line D				X (Drywell)	
MS-S-532	M.S. Relief Line D				X (Drywell)	
MS-S-533	M.S. Relief Line D				X (Drywell)	
MS-S-534	M.S. Relief Line D				X (Drywell)	

## BASES:

### 3.6.I & 4.6.I

#### SHOCK SUPPRESSORS (SNUBBERS)

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient, while allowing normal thermal motion during startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of a seismic or other event initiating dynamic loads. It is therefore required that all snubbers required to protect the primary coolant system and all other safety related systems or components be operable during reactor operation.

The visual inspection frequency is based on maintaining a constant level of snubber protection to systems. The cumulative number of inoperable snubbers detected during any inspection interval is the basis for establishment of the subsequent inspection interval and the existing inspection interval should remain in effect until its completion.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable.

Generically susceptible snubbers are those which are of a specific make or model and have the same design features directly related to rejection of the snubber by visual inspection, and are exposed to the same environmental conditions such as temperature, radiation, and vibration.

When a snubber is found inoperable, an engineering evaluation is initiated, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. Initiating this evaluation within 72 hours ensures that prompt corrective action will be afforded.

Hydraulic snubbers and mechanical snubbers may each be treated as a different entity for the above surveillance programs.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc...). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation. Due to the number and complexity of the relevant interacting factors necessary to develop a comprehensive Service Life Program, this program shall become effective July 1, 1982.



### 3. Special Reports

Special reports shall be submitted as indicated in Table 6.9.1.

### 6.10 RECORD RETENTION

A. The following records shall be retained for at least five years:

1. Records of facility operation covering time interval at each power level.
2. Records of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
3. Reportable Occurrence Reports.
4. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
5. Records of reactor tests and experiments.
6. Records of changes made to Operating Procedures.
7. Records of radioactive shipments.
8. Records of sealed source leak tests and results.
9. Records of annual physical inventory of all source material of record.

B. The following records shall be retained for the duration of the Operating License:

1. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
2. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
3. Records of facility radiation and contamination surveys.
4. Records of radiation exposure for all individuals entering radiation control areas.
5. Records of the service lives of all hydraulic and mechanical snubbers listed on Tables 3.6.I(a) and 3.6.I(b) including the date at which the service life commences and associated installation and maintenance records.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-293

BOSTON EDISON COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 60 to Facility Operating License No. DPR-35 issued to Boston Edison Company (the licensee) which revised the Technical Specifications for operation of the Pilgrim Nuclear Power Station (the facility) located near Plymouth, Massachusetts. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications pertaining to inservice surveillance and operability requirements of mechanical and hydraulic snubbers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since it does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated October 15, 1981, (2) Amendment No. 60 to License No. DPR-35, and (3) the Commission's letter to the licensee dated April 12, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Plymouth Public Library, North Street, Plymouth, Massachusetts 02360. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 12th day of April 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing