

Docket No. 50-293

MARCH 9 1978

Boston Edison Company
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Gentlemen:

The Commission has issued the enclosed Amendment No. 30 to Operating License No. DPR-35 for the Pilgrim Nuclear Power Station. This amendment changes the Technical Specifications in response to your request dated October 28, 1977 and our letter dated August 12, 1977.

The amendment consists of administrative changes to the Technical Specifications to:

1. delete the requirement for an Annual Operating Report, while retaining the specific requirement for an Annual Report of Occupational Exposure,
2. modify the submittal date for the Monthly Operating Report to the 15th instead of the 10th of the month following the calendar month covered by the report,
3. incorporate editorial and administrative items relating to previous errors, title changes, and deletion of completed items, and
4. delete the Respiratory Protection Program based on your compliance with 10 CFR 20.103, and our letter dated August 12, 1977.

Our review of your applications resulted in minor modifications to your proposals. These have been discussed with and agreed to by your staff.

Your proposed changes to specifications 6.5.B.2 and 6.5.B.10.c are not acceptable as written and we will defer action on these two proposed changes pending additional discussion with your staff.

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

[Signature]
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

ORB-36
G. Lear
3/7/78

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Enclosures:

1. Amendment No. 30 to
License No. DPR-35
2. Safety Evaluation
3. Notice

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Boston Edison Company (the licensee) dated October 28, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

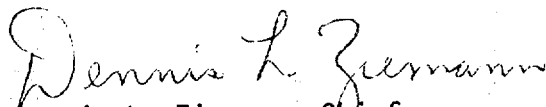
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-35 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 30, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 9, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 30

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

The following changes relate to the Appendix A portion of the Pilgrim Unit No. 1 Technical Specifications. The changed areas on the revised pages are shown by a marginal line.

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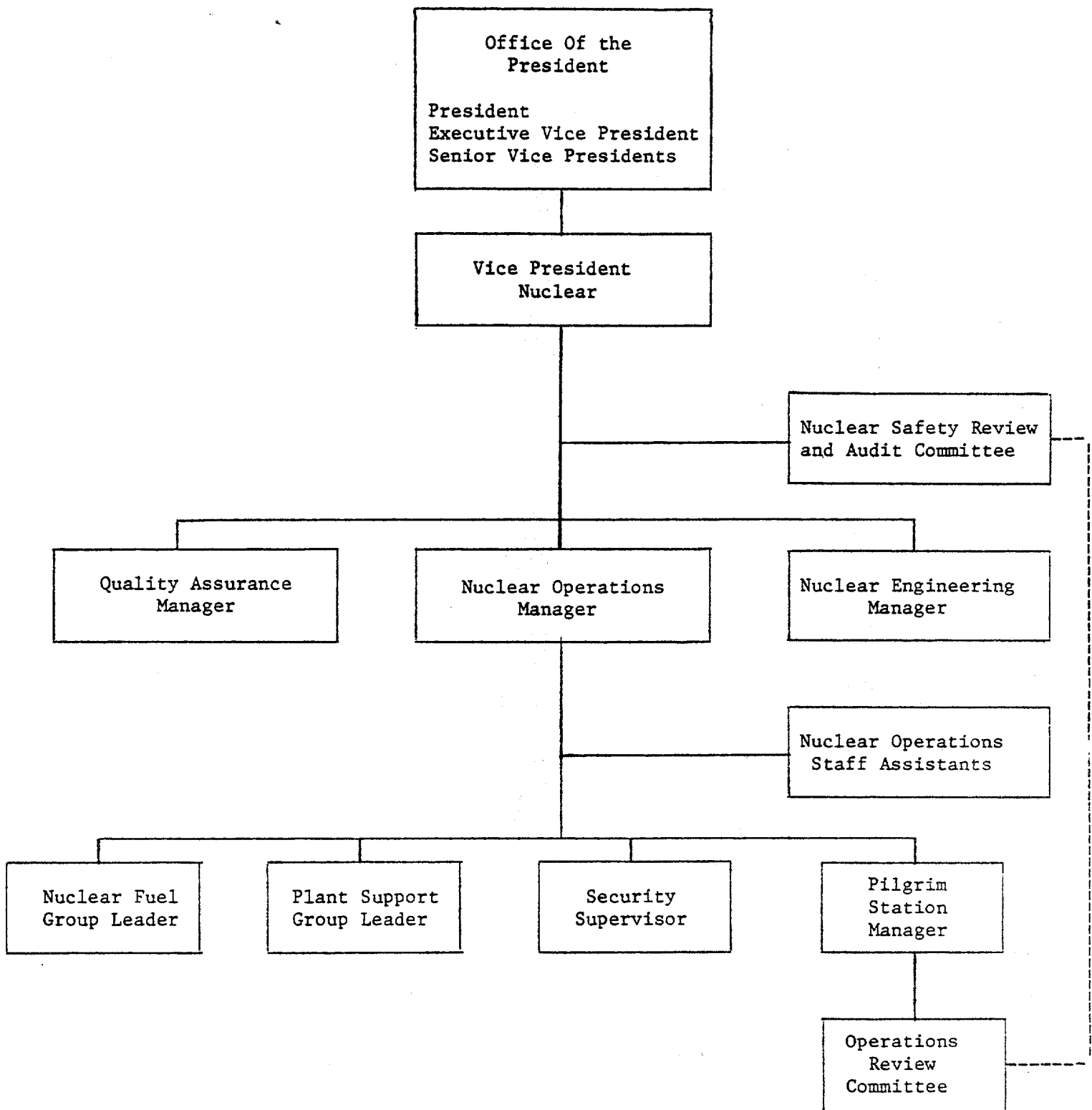


Figure 6.2.1

Boston Edison Company -- Organization for Operation
of Pilgrim I

6.5 REVIEW AND AUDIT

A. OPERATIONS REVIEW COMMITTEE (ORC)

1. FUNCTION

The ORC shall function to advise the Pilgrim Station Manager on all matters related to nuclear safety.

2. COMPOSITION

The ORC shall be composed of the:

Chairman:	Station Manager
Member:	Methods, Compliance & Training Group Leader
Member:	Chief Operating Engineer
Member:	Chief Technical Engineer
Member:	Chief Maintenance Engineer

3. ALTERNATES

Alternate members shall be appointed in writing by the ORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in an ORC quorum at any one time.

4. MEETING FREQUENCY

The ORC shall meet at least once per calendar month and as convened by the ORC Chairman.

5. QUORUM

A quorum of the ORC shall consist of the Chairman and two members including alternates.

6. RESPONSIBILITIES

The ORC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto that affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Nuclear Operations Manager and to the NSRAC Chairman.

- g. All events which are required by regulation or Technical Specifications to be reported to the NRC in writing within 24 hours.
- h. Any other matter involving safe operation of the nuclear plant which NSRAC deems appropriate for consideration or which is referred to NSRAC by the onsite operating organization or by other functional organizational units within Boston Edison.
- i. Reports and meeting minutes of the Operations Review Committee.

8. AUDITS

Audits of facility activities shall be performed under the cognizance of the NSRAC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions required by deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Emergency Plan and implementing procedures at least once per two years.
- f. The Station Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the NSRAC or the Vice President-Nuclear.
- h. The Fire Protection Program and implementing procedures at least once per two years.

9. AUTHORITY

The NSRAC shall report to and advise the Vice President-Nuclear on those areas of responsibility specified in Section 6.5.B.7 and 6.5.B.8.

10. RECORDS

Records of NSRAC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NSRAC meeting shall be prepared, approved and forwarded to the Vice President-Nuclear, NSRAC members, and others the Chairman may designate, within 14 days following each meeting.

- b. Reports of reviews encompassed by Section 6.5.B.7 e, f, g and h above, shall be prepared, approved and forwarded to the Vice President-Nuclear, with a copy to the Pilgrim Station Manager, within 21 days following the completion of the review.
- c. Audit reports encompassed by Section 6.5.B.8 above shall be forwarded to the Vice President-Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

The following actions shall be taken in the event of a reportable occurrence:

- A. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- B. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the ORC and submitted to the NSRAC Chairman and the Nuclear Operations Manager.

6.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- A. The provisions of 10 CFR 50.36(c) (1) (i) shall be complied with immediately.
- B. The Safety Limit Violation shall be reported to the Commission, the Nuclear Operations Manager and to the NSRAC Chairman immediately.
- C. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the ORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- D. The Safety Limit Violation Report shall be submitted to the Commission, the NSRAC Chairman and the Nuclear Operations Manager within 14 days of the violation.

6.8 PROCEDURES

- A. Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7 - 1972 and Appendix "A" of USNRC Regulatory Guide 1.33, except as provided in 6.8.B and 6.8.C below.

- B. Each procedure a administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the ORC and approved by the Station Manager prior to implementation and periodically as set forth in station procedures.
- C. Temporary changes to procedures of 6.8.A above may be made provided:
 - 1. The intent of the original procedure is not altered.
 - 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.
 - 3. The change is documented, reviewed subsequently by the ORC, and approved by the Station Manager within 7 days of implementation.
- D. Written procedures to implement the Fire Protection Program shall be established, implemented and maintained.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

A. Routine Reports

- 1. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

2. Monthly Operating Report. Routine reports of operating statistics, shutdown experience and forced reductions in power shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, US Nuclear Regulatory Commission, Washington, DC 20555, with a copy to the appropriate Regional Office, to arrive no later than the 15th of each month following the calendar month covered by the report.

The Monthly Operating Report shall include a narrative summary of operating experience that describes the operation of the facility, including safety-related maintenance, for the monthly report period.

3. Occupational Exposure Tabulation. A tabulation of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (including a description), waste processing, and refueling shall be submitted on an annual basis. This tabulation supplements the requirements of 20.407 of 10 CFR 20. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

B. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

1. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 6.9.B.1.e, 6.9.B.1.f, or 6.9.B.2.a below.

- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 6.9.B.2.b below.

3. Special Reports

Special reports shall be submitted as indicated in Table 6.9.1.

6.10 RECORD RETENTION

A. The following records shall be retained for at least five years:

1. Records of facility operation covering time interval at each power level.
2. Records of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
3. Reportable Occurrence Reports.
4. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
5. Records of reactor tests and experiments.
6. Records of changes made to Operating Procedures.
7. Records of radioactive shipments.
8. Records of sealed source leak tests and results.
9. Records of annual physical inventory of all source material of record.

B. The following records shall be retained for the duration of the Operating License:

1. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
2. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
3. Records of facility radiation and contamination surveys.
4. Records of radiation exposure for all individuals entering radiation control areas.

TABLE 6.9.1

<u>Area</u>	<u>Reference</u>	<u>Submittal Date</u>
a. Secondary Containment Leak Rate Testing (1)	4.7.C.c	Upon completion of each test (2)
b. In-service Inspection Evaluation	4.6.G.	Five years after commercial operation
c. (Deleted)		
d. Gross Gaseous Release 0.05 Ci/sec for 48 Hours	4.8.B.	Ten days after the release occurs

- NOTES:
1. Each integrated leak rate test of the secondary containment shall be the subject of a summary technical report. This report should include data on the wind speed, wind direction, outside and inside temperatures during the test, concurrent reactor building pressure, and emergency ventilation flow rate. The report shall also include analyses and interpretations of those data which demonstrate compliance with the specified leak rate limits.
 2. The report shall be submitted within 90 days after completion of each test. Test periods shall be based on the commercial service date as the starting point.

5. Records of gaseous and liquid radioactive material released to the environs.
6. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.
7. Records of training and qualification for current members of the plant staff.
8. Records of in-service inspections performed pursuant to these Technical Specifications.
9. Records of Quality Assurance activities required by the QA Manual.
10. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
11. Records of meetings of the ORC and the NSRAC.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 (Deleted)

6.13 HIGH RADIATION AREA

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, an acceptable alternate to controlling access to a high radiation area is as follows:

- A. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose in the area.
- B. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.A above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Watch Engineer on duty.

6.14 FIRE PROTECTION PROGRAM

The following inspections and audits shall be performed:

1. An independent fire protection inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm.
2. An inspection and audit by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. DPR-35
BOSTON EDISON COMPANY
PILGRIM UNIT 1
DOCKET NO. 50-293

Introduction

By application dated October 28, 1977, Boston Edison Company (BECo) proposed changes to the Technical Specifications for Pilgrim Nuclear Power Station Unit No. 1. By our letter dated August 12, 1977, we advised BECo of our intention to delete the Technical Specification requirements for a Respiratory Protection Program. Therefore, the Technical Specifications for Pilgrim Nuclear Power Station Unit No. 1 would be modified to:

1. delete the requirement for an Annual Operating Report, while retaining the specific requirement for an Annual Report of Occupational Exposure,
2. modify the submittal date for the Monthly Operating Report to the 15th instead of the 10th of the month following the calendar month covered by the report,
3. incorporate editorial and administrative items relating to previous errors, title changes, and deletion of completed items, and
4. delete the Respiratory Protection Program based on BECo compliance with 10 CFR 20.103, and our letter dated August 12, 1977.

Our review of the BECo application dated October 28, 1977, resulted in minor modifications to BECo's proposals. The changes have been discussed with and agreed to by the BECo staff.

Discussion and Evaluation

1. Annual and Monthly Operating Reports

Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", is the basis for reporting requirements found in Technical Specifications today. When these Technical Specifications were issued, we requested that licensees use the formats in the guide for the Licensee Event Report (LER) and Monthly Operating Report. After two years of experience with the existing reporting requirements, we reviewed the scope of information that licensees are required to submit in the LER, Annual Operating Report, Monthly Operating Report and Startup Report. Based on our review of LER's we developed a modified format for the LER to make the document more useful for evaluation purposes. By letter dated August 12, 1977, we informed BECo of the new LER format and requested that they use it.

From our review of the various licensee reports, we determined that much of the information found in the Annual Operating Report either is addressed in the LER's or Monthly Operating Reports, which are submitted in a more timely manner, or could be included in these reports with only a slight augmentation of the information already supplied. Therefore, we concluded that the Annual Operating Report could be deleted as a Technical Specification requirement if certain additional information were provided in the Monthly Operating Reports. As a result, we advised BECo by letter dated September 19, 1977, that a revised, improved format for the Monthly Operating Reports was available and requested that they use it. In addition, BECo was informed that if they agreed to use the revised format they should submit a change request to delete the requirement for an Annual Operating Report except that occupational exposure data must still be submitted. By letter dated October 28, 1977, BECo proposed deleting the Annual Operating Report, continuing the annual occupational exposure data report, and modifying the Monthly Operating Report. The report which tabulates occupational exposure on an annual basis is needed and has been retained in the BECo proposal. The information concerning forced reductions in power and outages will be supplied in the revised Monthly Operating Reports and the narrative summary of operating experience will be provided on a monthly basis in the Monthly Operating Report rather than annually. BECo has committed to use of the revised Monthly Operating Report as we requested. We agree that all needed information will be provided and deletion of the Annual Operating Report is acceptable. We also believe that an additional five days would be needed for compilation and submittal of the more detailed Monthly Report. We have modified the specifications accordingly. BECo staff agrees to this change.

2. Editorial and Administrative Changes

Miscellaneous Technical Specification changes have been proposed by BECo. These changes are:

- (1) Figure 6.2.1 - Changes title of "Nuclear Licensing Administrator - Operations" to "Plant Support Group Leader".
- (2) Specification 6.5.A.2 - Change the title "Management Systems Coordinator" to "Methods, Compliance and Training Group Leader".
- (3) Specification 6.5.B.2 - A typographical error in the word "President" is corrected.
- (4) Specification 6.5.B.2 - Removes specific educational and experience requirements for Nuclear Safety Review and Audit Committee members.
- (5) Specification 6.5.B.8 - Changes the letter designation of one section from "j" to "g" to correct a typographical error and change the title of the "Vice President - Operations and Engineering" to "Vice President - Nuclear".
- (6) Specification 6.5.B.10.b - Changes the title of the "Division Head" to "Station Manager" to reflect usage in other sections of the Technical Specifications.
- (7) Specification 6.5.B.10.c - Eliminates the requirement to provide the Vice President - Nuclear with copies of all NSRAC audits.
- (8) Specification 6.6.B - Adds the word "Chairman" after "NSRAC" to correct a typographical omission.
- (9) Specification 6.8.D - Deletes the requirement to implement certain procedures. These procedures have been implemented and the specification is no longer applicable.
- (10) Specification 6.9.C.2 - Deletes two special environmental reports that are no longer required. This corrects an omission that occurred in Amendment No. 23.
- (11) Table 6.9.1 Item b - Changes the submittal date of BECo's Inservice Inspection Evaluation from "Five years after commercial operation" to "Five years after initial

criticality". This change in reporting requirement provides conformance with the Basis for Specification 4.6.G which specifies the need for the evaluation.

- (12) Table 6.9.1 Item (c) deletes the requirement to submit the report that was submitted on June 18, 1976.

Proposed changes (4) and (7) above have not been resolved acceptably at this time. Action on these two items will be deferred pending additional discussion with BECo.

The remaining 10 proposed changes are administrative in nature relating to title changes, correction of typographical errors, and deletion of completed items. We have reviewed these changes and find that they are administrative in nature, do not adversely affect the operation of the Pilgrim Station, and will not result in any change in the mode of operation previously approved. Therefore, we conclude that these ten changes are acceptable.

3. Respiratory Protection Program

By letter dated August 12, 1977, we advised BECo of an amendment to 10 CFR 20.103 published by the Commission on November 29, 1976 to become effective on December 29, 1976. One effect of this is that in order to receive credit for limiting the inhalation of airborne radioactive material, respiratory protection equipment must be used as stipulated in Regulatory Guide 8.15. Another requirement of the amended regulation is that licensees authorized to make allowance for use of respiratory protective equipment prior to December 29, 1976, must bring the use of their respiratory protective equipment into conformance with Regulatory Guide 8.15 by December 29, 1977.

The respiratory protective program described in Section 6.12 of the Pilgrim Technical Specifications differs from that stipulated in Regulatory Guide 8.15. In view of the provisions of Section 6.11 of the Technical Specifications, which require conformance with 10 CFR 20, the fact that 10 CFR 20.103 no longer requires specific authorization to employ respiratory protective equipment, and the revocation provisions of Technical Specification 6.12.C, we conclude that merely deleting Section 6.12 is appropriate. The BECo staff is in agreement.

Accordingly, pursuant to 10 CFR 20.103(c) and (f), if BECo desires to receive credit for use of respiratory protective equipment at Pilgrim after December 28, 1977, such use must be as stipulated in Regulatory Guide 8.15 rather than as was specified in deleted Technical Specification 6.12. Based on the revocation provision of the specification on respiratory protection noted above and in the absence of prior written objection from BECo, we have deleted Specification 6.12 in its entirety from the Technical Specifications for the facility.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 9, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-293

BOSTON EDISON COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 30 to Facility Operating License No. DPR-35, issued to Boston Edison Company (the licensee), which revised the Technical Specifications for operation of the Pilgrim Nuclear Power Station (the facility) located near Plymouth, Massachusetts. The amendment is effective as of its date of issuance.

The amendment consists of administrative changes to the Technical Specifications to: (1) delete the requirement for an Annual Operating Report, while retaining the specific requirement for an Annual Report of Occupational Exposure, (2) modify the submittal date for the Monthly Operating Report to the 15th instead of the 10th of the month following the calendar month covered by the report, (3) incorporate editorial and administrative items relating to previous errors, title changes, and deletion of completed items, and (4) delete the Respiratory Protection Program based on your compliance with 10 CFR 20.103 since this item is now covered by 10 CFR §20.103 of Part 20 of the Commission's regulations.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made

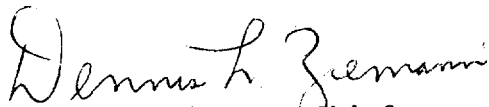
appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 28, 1977, (2) Amendment No. 30 to License No. DPR-35, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Plymouth Public Library on North Street in Plymouth, Massachusetts 02360. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 9th day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors