UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
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ATLANTA, GEORGIA 30303-8931

July 6, 2001

Westinghouse Electric Company
ATTN: Mr. R. Monley, Manager
Columbia Plant
Nuclear Fuel Business Unit
Drawer R
Columbia, SC 29250

SUBJECT: NRC INSPECTION REPORT NO. 70-1151/2001-05

Dear Mr. Monley:

This letter refers to the inspection conducted on June 4-7, 2001, at the Columbia Plant. The purpose of the inspection was to determine whether activities authorized by the license were conducted safely and in accordance with NRC requirements. At the conclusion of the inspection, the findings were discussed with those members of your staff identified in the enclosed report.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observation of activities in progress.

Based on the results of the inspection, violations or deviations were not cited.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

Should you have any questions concerning this letter, please contact us.

Sincerely,

/RA/

Edward J. McAlpine, Chief
Fuel Facilities Branch
Division of Nuclear Materials Safety

Docket No. 70-1151
License No. SNM-1107

Enclosure: (See Page 2)

Enclosure: NRC Inspection Report

cc w/encl:
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U.S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket No.:	70-1151
License No.:	SNM-1107
Report No.:	70-1151/2001-05
Licensee:	Westinghouse Electric Company
Facility:	Columbia Fuel Fabrication Facility Columbia, SC 29250
Date:	June 4-7, 2001
Inspector:	W. B. Gloersen, Senior Fuel Facilities Inspector
Approved by:	E. J. McAlpine, Chief Fuel Facilities Branch Division of Nuclear Materials Safety

Enclosure

EXECUTIVE SUMMARY

Commercial Nuclear Fuel Division
NRC Inspection Report 70-1151/2001-05

The focus of this routine, announced inspection was the observation and evaluation of the licensee's programs for transportation of radioactive materials, radioactive solid waste management, and management organization and controls. The report includes inspection efforts of one regional inspector. The inspection identified the following aspects of the licensee programs as outlined below:

Management Organization and Controls

- The changes to the shipping and transportation organization enhanced the program in that managers, technical, and engineering staff were not assigned collateral duties in other departments. The managers met the minimum education and experience requirements specified in the license application (Section 2.a).

- Internal Audits that pertained to shipping and transportation were acceptable and there was a system in place for tracking corrective actions to completion (Section 2.b).

- The quality assurance program did not ensure that licensee audits would focus on compliance with the waste classification and characterization requirements of 10 CFR 61.55 and 61.56 (Section 2.b).

Transportation

- Management approved procedures were acceptable to ensure that the MCC fuel shipping packages would be properly loaded in accordance with the NRC Certificate of Compliance (CoC) (Section 3.a).

- The maintenance activities and procedures associated with the refurbishment of the MCC fuel assembly shipping containers were acceptable. The package refurbishment area was clean, orderly, and the operators were knowledgeable of their procedures and craft (Section 3.b).

- The CoC for the NRC approved shipping containers used to ship radioactive materials were well organized. The waste shipping manifests were complete and provided an acceptable level of information in the shipping papers to determine the quantities of individual radionuclides shipped. Shipping records for radioactive material shipments were complete and the information supplied on the shipping papers was appropriate (Section 3.c).

- The Uranyl Nitrate crystal package receipt, opening, material transfer, and container survey process was conducted efficiently, safely, and in accordance with established procedures. The radioactive material unloading work area was well laid out and included ventilated enclosures to control airborne radioactivity (Section 3.d).

- One violation of minor safety significance involving the package effectiveness for the MCC-5 fuel assembly shipping containers' bottom hold down assembly was identified by the licensee. Two non-cited violations were identified for the failure to comply with the terms and conditions of Certificate of Compliance No. 9239 for the MCC-5 shipping container and the failure to properly install and secure the closure device of a packaging containing fissile material (Section 3.e).

Radioactive Solid Waste Management

- The licensee was effectively reducing the volume of LLRW being sent for offsite disposal (Section 4.a).

- Solid waste streams were appropriately classified as Class A waste in stable form and the waste met the applicable minimum waste characterization requirements specified in 10 CFR 61.56(a) (Section 4.b).

Attachment:
Persons Contacted
Lists of Items Opened, Closed, and Discussed
List of Acronyms

REPORT DETAILS

1. Summary of Plant Status

This report covered a four day period. The conversion lines were operating below normal capacity. Pellet and fuel assembly production remained near capacity during the week.

2. Management Organization and Controls (Inspection Procedures (IPs) 88005, 84850, and 86740)

a. Organizational Structure

(1) Inspection Scope

The inspector examined the licensee's organizational structure for the radioactive materials shipment program. Specifically, the inspector reviewed any organizational changes, structural changes, and changes in personnel responsibilities and functions that occurred since the last inspection.

(2) Observations and Findings

The inspector discussed with the licensee the organizational changes and changes in personnel responsibilities and functions that occurred since the last inspection of the shipping and transportation program. The organizational structure changed in November 2000. The new organization included engineers and technical staff that no longer had collateral duties in other departments. The licensing and logistical function, which included nuclear criticality safety and container design management, of the shipping program reported directly to the Nuclear Fuel Transport Manager of British Nuclear Fuels Limited (BNFL). The Nuclear Fuel Transport Manager for Columbia Operations also reported to the BNFL Nuclear Fuel Transport Manager. Shipping, receiving, hazardous materials, and container engineering reported to the Nuclear Fuel Transport Manager for Columbia Operations. At present, the BNFL Nuclear Fuel Transport Manager reported directly to the Vice President (VP) of European Fuel and indirectly to the VP of US Manufacturing.

The solid waste management function reported directly to the Chemical Process Engineering Manager, who in turn reported to the Technical Services Manager.

Although the shipping and transportation functions were not specifically detailed in Chapter 2 , Management Organization, of the license application, the managers met the minimum education and experience requirements specified in the license application.

(3) Conclusion

The changes to the shipping and transportation organization enhanced the program in that managers, technical, and engineering staff were not assigned collateral duties in other departments. The managers met the minimum education and experience requirements specified in the license application.

b. Internal Reviews, Audits, and Self Assessments

(1) Inspection Scope

The inspector examined the licensee's periodic programmatic assessments established by the licensee to ensure that safety commitments in the license were being met.

(2) Observations and Findings

The inspector selected internal audits that pertained to shipping, transportation, and solid waste management. The inspector verified that the licensee had a system in place for tracking corrective actions to completion. The inspector reviewed internal audit WEC-01-02, dated March 2001 and two low level radioactive waste (LLRW) audits conducted in 1999 and 2000. The LLRW audit addressed waste generation, segregation, decontamination, sorting, packaging, surveying, labeling, shipping, and record keeping. The inspector discussed with the licensee the need for the Westinghouse Quality Assurance (QA) program to ensure compliance with the waste classification and characterization requirements of 10 CFR 61.55 and 61.56. The nature and scope of the QA program will vary depending on the changes and complexity of the waste stream. The licensee acknowledged the inspector's comments and indicated that this area would be focused on during subsequent audits.

The inspector reviewed two self-assessments performed by the licensee. Self-assessments at Westinghouse were not periodic, programmatic reviews, but were rather investigations triggered by events, identified issues, or other problems. Section 2.1 of procedure CA-203, Self Assessment Guidelines, Revision (Rev.) 0, March 23, 2000, states that self-assessments shall be conducted at the direction of management to review and evaluate an area of interest where compliance to the quality program requirements or performance in general may be improved. The inspector reviewed the following two self-assessments: (1) Inappropriate Information for Pellet Shipment to Hematite, dated February 16, 2001; and (2) MCC-5 Shipping Container Bottom Nozzle Hold Down Assembly, dated December 4, 2000. The focus of the self-assessments was appropriate and the corrective actions were adequately addressed.

(3) Conclusion

Internal Audits that pertained to shipping and transportation were acceptable and there was a system in place for tracking corrective actions to completion. The quality assurance program did not ensure that licensee audits would focus on compliance with the waste classification and characterization requirements of 10 CFR 61.55 and 61.56.

3. Transportation (IP 86740, 84850)

The inspector reviewed the licensee's program for the shipment of radioactive materials, to determine whether the licensee had established and was maintaining an effective management-controlled program, to ensure radiological and nuclear safety in the receipt, packaging, delivery to a carrier of licensed radioactive materials, and to determine whether transportation activities were in compliance with the applicable NRC

and Department of Transportation (DOT) transport regulations noted below. During the inspection, transportation and radiation protection activities associated with fissile material shipments, including procedural guidance, quality control activities, record completeness, and radiation surveys conducted in accordance with 10 CFR 20, 10 CFR 71, and 49 CFR Parts 171-178 were reviewed.

10 CFR 71.5(a) requires that licensees who transport licensed material outside the confines of its plant or other place of use, or who delivers licensed material to a carrier for transport, shall comply with the applicable requirements of the regulations appropriate to the mode of transport of the DOT in 49 CFR Parts 170 through 189.

a. Preparation and Delivery of Completed Packages for Shipment

(1) Inspection Scope

The inspector examined the licensee's written procedures related to the preparation and delivery of completed packages for shipment of licensed material.

(2) Observations and Findings

The inspector verified that the licensee had procedures for the preparation of shipping packages and delivery of the model MCC packages to the carrier for the shipment of unirradiated fuel assemblies. The inspector reviewed selected portions of the following procedures:

- Maintenance Operating Procedure (MOP) -730703, Prepare Container for Loading Fuel Assemblies, Rev. 29, dated May 17, 2001

- MOP-730713, Load Fuel Assembly into Model MCC Shipping Containers, Rev. 63, dated May 17, 2001

The procedures incorporated check-off lists to ensure certain loading operations were performed in the appropriate sequence and in accordance with the Certificate of Compliance (CoC).

(3) Conclusion

The licensee had acceptable management approved procedures to ensure that the MCC fuel shipping packages would be properly loaded in accordance with the NRC CoC.

b. Periodic Maintenance of Packagings

(1) Inspection Scope

The inspector reviewed the licensee’s program for periodic maintenance and refurbishment of fuel assembly shipping packages.

(2) Observations and Findings

Chapter 8 of the license application for the MCC shipping container specified the acceptance tests, maintenance program, and re-certification program. In addition to the requirements specified in the NRC CoC No. 9239 for fuel shipping containers, the requirements for routine determinations specified in 10 CFR 71.87 and 49 CFR 173.475 were applicable.

The inspector verified that the licensee had procedures for the periodic maintenance of the model MCC shipping packages. The inspector reviewed selected portions of the following procedure:

- MOP-755707, Refurbishing Shipping Container, Revision 42, May 17, 2001

For reusable NRC-certified packagings for fuel assemblies, the inspector examined the licensee's procedures and records for refurbishment and maintenance and verified that before re-use, all of the periodic maintenance required by the CoC (and Chapter 8 of the application) had been incorporated into the procedures and had been performed. During the inspection, the inspector observed maintenance and refurbishment activities on four Model MCC fuel shipping containers, including the gasket inspection. The operators used the checklist in form CF-75B-002, Fuel Assembly Shipping Container Inspection Checklist to ensure that the required maintenance was performed. During this inspection, there were no MCC package re-certifications occurring. The inspector observed that the package refurbishment area was clean, orderly, and the operators were knowledgeable of their procedures and craft.

(3) Conclusion

The maintenance activities and procedures associated with the refurbishment of the MCC fuel assembly shipping containers were acceptable. The package refurbishment area was clean, orderly, and the operators were knowledgeable of their procedures and craft.

c. Records

(1) Inspection Scope

The inspector reviewed selected NRC CoCs for packages used by the licensee to ship fissile materials. 10 CFR 71.12(a) requires, in part, that a general license is issued to any licensee of the Commission to deliver to a carrier for transport, licensed material in a package for which a license, CoC, or other approval has been issued by the NRC. The general license specified in 10 CFR 71.12 applies to a licensee who (1) has a copy of the CoC and the drawings and other documents referenced in the approval relating to the use and maintenance of the packaging and to the actions to be taken prior to shipment; and (2) complies with the terms and conditions of the license, certificate, or other approval, as applicable, and the applicable requirements of Subparts A, G, and H of this part.

The inspector also reviewed shipping records for radioactive materials shipments and waste shipments.

(2) Observations and Findings

The inspector verified that the licensee had copies of the following NRC CoCs for packages used by the licensee to ship licensed material:

- NRC CoC 9239, Rev. 11, USA/9239/AF, Model Nos. MCC-3, MCC-4, and MCC-5

- NRC CoC 9203, Rev. 11, USA/9203/AF, Model DHTF

The inspector also verified that the licensee had registered with the NRC as a user of the NRC certified packages. The licensee's record maintenance system for the CoC's as well as the transport package Safety Analysis Reports was well organized.

The inspector noted that the licensee had requested that NRC CoC 6078 for shipping package model Nos. 927A1 and 927C1 be transferred from CE Nuclear Power LLC to Westinghouse Electric Company. The licensee received the authorization to transfer the CoC on October 10, 2000. The licensee accepted the responsibility for the maintenance of the certificates, the safety analyses for package designs, and the quality assurance records in accordance with the requirements of 10 CFR 71.91(c). The licensee plans to use the 927A1 and 927C1 shipping containers to ship Combustion Engineering fuel assemblies later in 2001. At the time of this inspection, the licensee was in the process of developing operating procedures, maintenance procedures and an operator training program before first use of the shipping containers.

The inspector also reviewed selected records for radioactive materials and waste shipments for the period March through May 2001. The inspector reviewed manifest records of selected shipments of radioactive waste. The manifests were complete and met the applicable requirements of Appendix G to 10 CFR Part 20. The inspector also verified that the licensee had a procedure and program in place to track waste. The inspector also reviewed licensee transportation activities regarding shipments of unirradiated fuel, uranium dioxide (UO_2) powder, and UO_2 pellets. Selected records covering the period noted above for those consignments were reviewed in detail. The inspector reviewed and discussed the documentation used, and subsequently maintained in the licensee's records for each radioactive material shipment, including, the Bill of Lading, Radioactive Material Shipment Record, Vehicle Inspection Report, Receipt and Loading Verification Checklist, Packing List (Fuel Assemblies/Component Assemblies), Fuel Shipment Information Form, Container Log Sheet, and Health Physics Survey Forms. The inspector noted that the shipping records were complete and the information supplied on the shipping papers was appropriate.

(3) Conclusion

The licensee maintained the CoC for the NRC approved shipping containers used to ship radioactive materials in a well organized manner. The waste shipping manifests were complete and provided an acceptable level of information in the shipping papers to

determine the quantities of individual radionuclides shipped. Shipping records for radioactive material shipments were complete and the information supplied on the shipping papers was appropriate.

d. Receipt and Opening of Packages

(1) Inspection Scope

The inspector reviewed the licensee's procedures and program for incoming radioactive material shipments to verify compliance with the applicable requirements in 10 CFR 20.1906 relating to the pickup from a carrier, receiving, and the safe opening of packages.

(2) Observations and Findings

On June 6, 2001, the inspector observed the licensee perform incoming receipt surveys of drums containing uranyl nitrate (UNH) crystals received from the Babcock and Wilcox Technology (BWXT) facility located in Lynchburg, Virginia. The inspector also observed that the UNH crystal package receipt, opening, material transfer, and container survey process was conducted efficiently, safely, and in accordance with established procedures. The inspector observed that the material unloading work area was well laid out and included ventilated enclosures to control airborne radioactivity. The operators were knowledgeable of their procedures. Radiation and contamination surveys were conducted appropriately.

(3) Conclusion

The UNH crystal package receipt, opening, material transfer, and container survey process was conducted efficiently, safely, and in accordance with established procedures. The radioactive material unloading work area was well laid out and included ventilated enclosures to control airborne radioactivity.

e. Reduction in Package Effectiveness Reports

(1) Inspection Scope

The inspector reviewed reduction in package effectiveness reports submitted by the licensee in accordance with 10 CFR 71.95 concerning self-identified violations of the CoC requirements for fuel assembly shipping containers.

(2) Observations and Findings

The inspector reviewed two reduction in package effectiveness reports submitted on August 7 and December 5, 2000, in accordance with the requirements specified in 10 CFR 71.95.

The 10 CFR 71.95 report, dated December 5, 2000, pertained to the following two issues related to the MCC-5 shipping container (NRC CoC 9239): (1) In 1995, the bottom nozzle hold down assembly for the MCC-5 shipping container, which was

classified as Category A safety-related part in the specifications, and identified as safety related on the equipment drawing, was fabricated in accordance with the requirements for a Category B safety related part. This problem was identified on or about November 28, 2000; and (2) It was also determined that the bottom hold down assembly was not included in the MCCL 501 series license drawing. The licensee initiated both immediate and long term corrective actions as noted in the licensee's December 5, 2000 letter to the NRC. The inspector verified that the corrective actions had been completed. The inspector in conjunction with the NRC's Spent Fuel Project Office, concluded that the licensee's report did not identify generic design or certificate issues, that the licensee's corrective actions were adequate, and that the safety significance was low. Based on the safety significance, problem (2) above was considered to be a minor violation. The non-repetitive, licensee-identified and corrected violation (problem (1)) pertaining to the fabrication of the bottom hold down assembly in accordance with the requirements of a Category B safety related part instead of a Category A safety related part is being treated as a non-cited violation (NCV), consistent with Section VII.B.1 of the NRC Enforcement Policy, and is identified as NCV 70-1151/2001-05-01: Failure to comply with the terms and conditions of Certificate of Compliance No. 9239 for the MCC-5 shipping container.

The 10 CFR 71.95 report, dated August 7, 2000, pertained to two DHTF containers that were used to ship fissile materials that had lid closure device bolts missing. The inspector verified that the corrective actions to prevent recurrence were completed. The safety significance for this problem was considered to be moderate due to several factors. First, the closure bolts were necessary to maintain the proper geometry of the containment contents following any impact to the package. Proper geometry is important for criticality control. Second, the extent of the condition was a concern. According to the 10 CFR 71.95 report to the NRC, the licensee reported that several packages in addition to the two that were discussed may have been shipped without some of the bolts, with as many as three out of eight bolts missing. In addition, 10 CFR 71.87(c) requires that each closure device of the packing, including any required gasket, be properly installed, secure, and free of defects. The cause was identified by Westinghouse as an improper translation of the CoC requirements and conditions into an operating procedure. However, based on the information provided in the 10 CFR 71.95 report, it appears that a questioning attitude by the operators could have found this discrepancy, even with out specific procedural guidance. The inspector concluded that the corrective actions as specified in the 10 CFR 71.95 report would prevent recurrence of the violation. Thus, this non-repetitive, licensee-identified and corrected violation is being treated as a NCV, consistent with Section VII.B.1 of the NRC Enforcement Policy, and is identified as NCV 70-1151/2001-05-02: Failure to properly install and secure the closure device of a packaging containing fissile material.

(3) Conclusion

One violation of minor safety significance involving the package effectiveness for the MCC-5 fuel assembly shipping containers' bottom hold down assembly was identified by the licensee. Two non-cited violations were identified for the failure to comply with the terms and conditions of Certificate of Compliance No. 9239 for the MCC-5 shipping container and the failure to properly install and secure the closure device of a packaging containing fissile material.

4. Radioactive Waste Management (IP 84850)

a. Waste Generation and Storage

(1) Inspection Scope

The inspector reviewed the licensee's generation and storage of LLRW, including management controls and surveys, and adequacy of the storage area.

(2) Observations and Findings

The inspector toured the LLRW storage and staging areas throughout the facility. With the exception of the dry active waste that could not be incinerated, the remainder of the LLRW was being staged for reprocessing so that the uranium could be recycled.

The inspector reviewed the licensee's performance in the volume reduction efforts for the disposal of LLRW. Since 1985, there has been a statistically significant reduction in the volume of LLRW generated for disposal. To illustrate this, in 1985, 23.5 E+03 ft^3 of LLRW was generated for disposal compared to approximately 1.0 E+03 ft^3 of LLRW generated in 2000. The licensee expects to generate approximately 0.4 E+03 ft^3 of LLRW in 2001. The inspector noted that the licensee has an aggressive program to reduce the volume of LLRW. These efforts can be attributed to controlling waste generated at the source and a combination of incineration of combustible wastes, recycling, cleaning of noncombustible wastes (metallic items) for possible free release, and compaction.

(3) Conclusion

The licensee was effectively reducing the volume of LLRW being sent for offsite disposal.

b. Waste Classification and Characterization

(1) Inspection Scope

The inspector reviewed the licensee's waste classification and characterization program. The inspector used the guidance in the NRC's Branch Technical Position paper on "Final Waste Classification and Waste Form" dated May 11, 1983.

(2) Observations and Findings

The requirements for waste classification and characterization are specified in 10 CFR 61.55 and 61.56, respectively. The licensee may rely on any of four methods to classify waste: (1) materials accountability; (2) classification by source; (3) gross radioactivity measurements; and (4) direct measurement of individual nuclides.

Westinghouse relied primarily on either of the first two methods to classify waste. The licensee's waste stream contained radionuclides of uranium and its decay products.

Uranium and its decay products were not listed in either Tables 1 or 2 of 10 CFR 61.55. The classification by sources method stipulates that the waste generated is classified and characterized through knowledge and control of the source of the waste. The source of the waste at Westinghouse was limited to the radionuclides of the uranium and actinium series. Therefore, all wastes generated by the fuel manufacturing process were designated as Class A. The inspector noted, however, that the licensee lacked a program document to ensure that LLRW would be properly classified and characterized in the event of a change in the waste stream. The licensee acknowledged the inspector's comment. This issue will be tracked as an inspector followup item ((IFI): 70-1151/2001-05-03, Document a program to ensure proper classification and characterization of LLRW).

(3) Conclusion

The licensee appropriately classified its solid waste streams as Class A waste in stable form and ensured that the waste met the applicable minimum waste characterization requirements specified in 10 CFR 61.56(a).

5. Exit Meeting

The inspection scope and results were summarized on June 7, 2001, with those persons indicated in the Attachment. The inspector described the areas inspected and discussed in detail the inspection results. Although proprietary documents and processes were reviewed during this inspection, the proprietary nature of these documents or processes has been deleted from this report. No dissenting comments were noted during the exit meeting.

ATTACHMENT

1. PARTIAL LIST OF PERSONS CONTACTED

Licensee Personnel (verify list and attendees)

*B. Bayley, Nuclear Fuel Transport Manager
*R. Ervin, Chemical Process Engineering Manager
R. Fischer, Senior Engineer
*N. Kent, Nuclear Fuel Transport Safety Engineer
S. McDonald, Environment, Health & Safety (EH&S) Manager
*J. Heath, Integrated Safety Engineering Manager
*B. Monley, Columbia Plant Manager
F. Moorer, Transportation Specialist
*S. Palmer, Container Engineer
J. Rankar, Associate Integrated Safety Engineer
*T. Ross, Transportation Manager
*W. Sillwell, Transportation Engineer
R. Williams, Advisory Engineer

*Denotes those present at the exit meeting on June 7, 2001.

2. INSPECTION PROCEDURES USED

IP 84850	Radioactive Waste Management
IP 86740	Inspection of Transportation Activities
IP 88005	Management Organization and Controls

3. LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Item Number	Status	Description
70-1151/01-05-01	Closed	NCV: Failure to comply with the terms and conditions of Certificate of Compliance No. 9239 for the MCC-5 shipping container (Section 3.e).
70-1151/01-05-02	Closed	NCV: Failure to properly install and secure the closure device of a packaging containing fissile material (Section 3.e).
70-1151/01-05-03	Opened	IFI: Document a program to ensure proper classification and characterization of LLRW (Section 4.b).

4. ACRONYMS and ABBREVIATIONS

BNFL	British Nuclear Fuels Limited
CFR	Code of Federal Regulations

CoC	Certificate of Compliance
DOT	Department of Transportation
IFI	Inspector Followup Item
IP	Inspection Procedure
LLRW	Low Level Radioactive Waste
MOP	Maintenance Operating Procedure
NCV	Non-cited Violation
NMSS	Nuclear Material Safety and Safeguards
NRC	Nuclear Regulatory Commission
QA	Quality Assurance
Rev.	Revision
SNM	Special Nuclear Material
UO_2	Uranium Dioxide
UNH	Uranyl Nitrate
VP	Vice President
WEC	Westinghouse Electric Company