

July 6, 2001

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing and Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 — REQUEST FOR ADDITIONAL INFORMATION
RE: PROPOSED LICENSE AMENDMENT REQUEST NO. 267, REVISION 2,
CONTAINMENT LEAKAGE RATE TESTING PROGRAM (TAC NO. MB1349)

Dear Mr. Young:

By letter dated March 7, 2001 (3F0301-05), you submitted an amendment application to revise the Crystal River Unit 3 (CR-3) Improved Technical Specification 5.6.2.20, "Containment Leakage Rate Testing Program" to allow a one-time interval extension for the Crystal River Unit 3 (CR-3) Type A, Integrated Leak Test (ILRT) for no more than 6 years. Florida Power Corporation (FPC) provided Supplemental Risk-Informed Information in Support of this request by letter dated April 25, 2001 (3F0401-11). FPC revised the extension interval to no more than 5 years and revised the risk-informed information by letter dated June 20, 2001(3F0601-06). The U.S. Nuclear Regulatory Commission staff is currently reviewing your request, and has determined that additional information is needed.

Because the containment inservice inspection (ISI) requirements of Title 10, Code of Federal Regulations, Section 50.55a, and leak rate testing requirements of Option B of Appendix J complement each other in ensuring the leak-tightness and structural integrity of the containment, the staff needs the following information to complete its review of the license amendment request.

1. None of the references describe (or summarize) the containment ISI program being implemented at CR-3. Please provide a description of the ISI methods that provide assurance that in the absence of an ILRT for 15 years, the containment structural and leak-tight integrity will be maintained.
2. Based on its review of the relief requests for CR-3 containment, the staff understands that CR-3 is using the 1992 Edition and the 1992 Addenda of Subsections IWE and IWL of the American Society of Mechanical Engineers Section XI Code (the Code). IWE-1240 requires licensees to identify the surface areas requiring augmented examinations. Please provide the locations of the containment liner surfaces which CR-3 have identified as requiring augmented examination and a summary of findings of the examinations performed.
3. For the examination of seals and gaskets, and examination and testing of bolts associated with the primary containment pressure boundary (Examination Categories E-D and E-G), CR-3 had requested relief from the requirements of the Code. As an

alternative, CR-3 plans to examine them during the leak rate testing of the primary containment. With the flexibility provided in Option B of Appendix J for Type B and Type C testing (as per Nuclear Energy Institute 94-01 and Regulatory Guide 1.163), and the extension requested in this amendment for Type A testing, please provide your schedule for examination and testing of seals, gaskets, and bolts that provide assurance regarding the integrity of the containment pressure boundary.

4. The stainless steel bellows have been found to be susceptible to trans-granular stress corrosion cracking, and the leakages through them are not readily detectable by Type B testing (see Information Notice 92-20). If applicable, please provide information regarding inspection and testing of the bellows at CR-3, and how such behavior has been factored into the risk assessment.
5. Inspections of some reinforced and steel containment systems have found degradation on the un-inspectable (embedded) side of the drywell steel shell and steel liner of the primary containment. These degradations cannot be found by visual (i.e., VT-3 or VT-1) examinations unless they are through the thickness of the shell or liner; or 100% of the un-inspectable surfaces are periodically examined by ultrasonic testing. Please provide information addressing how potential leakages under high pressures during core-damage accidents are factored into the risk assessment related to the extension of the ILRT.

For the staff to complete its review schedule, Mr. Ronald Bright of your staff stated that you would respond to this request prior to July 24, 2001. If there are any questions, please call me at 301-415-1437.

Sincerely,

/RA by D. Ashley for J. Goshen/

John M. Goshen, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

cc: See next page

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John M. Goshen, Project Manager, Section 2
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**CRYSTAL RIVER UNIT NO. 3
GENERATING PLANT**

cc:

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing
& Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Mr. R. Alexander Glenn
Associate General Counsel (MAC-BT15A)
Florida Power Corporation
P.O. Box 14042
St. Petersburg, Florida 33733-4042

Mr. Daniel L. Roderick
Plant General Manager
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Mr. Michael A. Schoppman
Framatome ANP
1911 North Ft. Myer Drive, Suite 705
Rosslyn, Virginia 22209

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, Florida 32399-1741

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. Joe Myers, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 34450-4245

Ms. Sherry L. Bernhoff
Manager Regulatory Affairs
Crystal River Nuclear Plant (NA2H)
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Senior Resident Inspector
Crystal River Unit 3
U.S. Nuclear Regulatory Commission
6745 N. Tallahassee Road
Crystal River, Florida 34428

Mr. Richard L. Warden
Manager Nuclear Assessment
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, Florida 34428-6708