

July 6, 2001

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON EXTENDED POWER
UPRATE - ARKANSAS NUCLEAR ONE, UNIT 2 (TAC NO. MB0789)

Dear Mr. Anderson:

By application dated December 19, 2000, you submitted an "Application for License Amendment to Increase Authorized Power Level." The requested power level increase is 7.5%.

The Nuclear Regulatory Commission staff has reviewed the information provided in the application. In order for the staff to continue its review, the staff requires a response to the enclosed request for additional information (RAI) on radiological dose assessment.

The contents of this RAI have been discussed with Mr. Dennis Boyd of your staff, and a response time frame of mid-August was agreed to. The staff appreciates your efforts in regard to this matter.

If for any reason this date becomes unreasonable, please contact me at your earliest opportunity.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-368

cc: See next page

Enclosure: RAI

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REQUEST FOR ADDITIONAL INFORMATION ON
RADIOLOGICAL DOSE ASSESSMENT
EXTENDED POWER UPRATE LICENSE AMENDMENT APPLICATION
ARKANSAS NUCLEAR ONE, UNIT 2

1. Please provide the calculations for the four events with dose increases (loss-of-coolant accident, steam generator tube rupture, control-element-assembly ejection, and fuel handling accident).
2. For those events with postulated dose increases in the replacement steam generator amendment request, the assumed control room envelope inleakage was 5000 cubic feet per minute (cfm). For those events (which are different events from those in the preceding sentence) with postulated dose increases for the power uprate amendment request, the assumed inleakage was 10 cfm. What is the basis for assuming different inleakage values for the same control room? If the value is to remain at 10 cfm for certain events, what is the basis for assuming that the Arkansas Nuclear One, Unit 2, control room envelope has such an integrity when, of the 25% of the plants that have tested their envelopes for integrity, none have demonstrated a value of 10 cfm and most have had values ranging from several hundred to several thousand cfm?

Arkansas Nuclear One

cc:

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