UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-8064

July 2, 2001

Mr. J. V. Parrish (Mail Drop 1023)
Chief Executive Officer
Energy Northwest
P.O. Box 968
Richland, Washington 99352-0968

SUBJECT: SUMMARY OF END-OF-CYCLE PERFORMANCE ASSESSMENT MEETING FOR THE COLUMBIA GENERATING STATION

Dear Mr. Parrish:

This refers to the public meeting conducted at the Walkley Room, Energy Northwest Multi-Purpose Facility, on June 21, 2001, between your staff and the NRC. The participants discussed the results of the NRC's end-of-cycle assessment of Columbia Generating Station's performance for the period April 2, 2000, through March 31, 2001.

The NRC presented the overall assessment results that were based on inspection findings and performance indicators. The presentation also included a brief overview of the reactor oversight process.

The attendance list, and the NRC prepared public meeting handouts are enclosed.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

Should you have any questions concerning this matter, we will be pleased to discuss them with you.

Sincerely,

Gregory A. Pick for

William B. Jones, Chief
Project Branch E
Division of Reactor Projects

Docket No: 50-397
License No: NPF-21

Enclosures: Attendance List
NRC End-of-Cycle Assessment Meeting, Columbia Generating Station, June 21,2001
Reactor Oversight Process Annual Assessment Meeting

cc w/enclosures:
Chairman
Energy Facility Site Evaluation Council
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Rodney L. Webring (Mail Drop PE08)
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Manager, Regulatory Affairs
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State Liaison Officer
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Office of the Governor
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Olympia, Washington 98504-3113

Lynn Albin
Washington State Department of Health
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Division of Radiation Protection
Department of Health
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Max E. Benitz, Jr., Chairman
Board of Benton County Commissioners
P.O. Box 190
Prosser, Washington 99350

Sue Miller, Chair
Board of Franklin County Commissioners
1016 North 4th Street
Pasco, Washington 99301

Electronic distribution from ADAMS by RIV:
Regional Administrator **(EWM)**
DRP Director **(KEB)**
DRS Director **(ATH)**
Senior Resident Inspector **(GDR)**
Branch Chief, DRP/E **(WBJ)**
Senior Project Engineer, DRP/E **(GAP)**
Section Chief, DRP/TSS **(PHH)**
RITS Coordinator **(NBH)**
B. Henderson, PAO **(BWH)**
C. A. Hackney, RSLO **(CAH)**
C. J. Gordon **(CJG)**
DRS Branch Chiefs **(GMG, JLS2, JLP)**
W. D. Travers, EDO **(WDT)**
W. M. Dean, Chief, NRR/DIPM/IIPB **(WMD)**
R. K. Frahm, PPR Program Manager, NRR/ILPB **(RKF)**
B. A. Boger, Associate Dir. for Inspection and Programs **(BAB2)**
B. W. Sheron, Associate Dir. for Project Licensing and Technical Analysis **(BWS)**
M. A. Satorius, Chief, Regional Operations Staff, OEDO **(MAS)**
S. Richards, NRR Project Director **(SAR)**
S. Dembek, Chief, Section 2, NRR/DLPM **(SXD)**
J. Cushing, NRR Project Manager **(JXC9)**

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GFLarkin	WBJones			
/RA/	***GAPick for***			
7/2/01	7/2/01			

OFFICIAL RECORD COPY T=Telephone E=E-mail F=Fax

ENCLOSURE 1

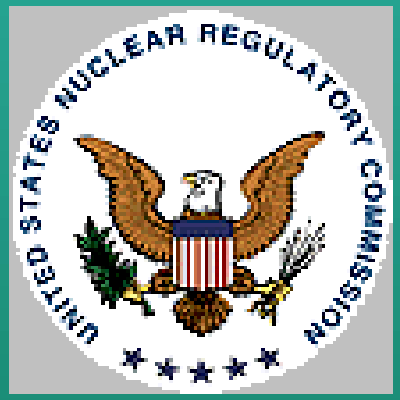
Attendance List

Licensee

S. Oxenford, Plant General Manager
D. Atkinson, Manager, Engineering
D. Coleman, Manager, Regulatory Affairs
L. Dovey, Communications Officer
D. Feldman, Manager, Operations
R. Graves, Executive Board Member
P. Inserra, Manager, Licensing
C. King, Manager, Design Engineering
J. Peters, Manager, Radiation Services
G. Smith, Vice President, Generation
R. Webring, Vice President, Operations Support
S. Wood, Manager, Chemistry

NRC

W. Jones, Chief, Reactor Projects Branch E
G. Replogle, Senior Resident Inspector
J. Melfi, Project Engineer
J. Moreno, Engineering Associate



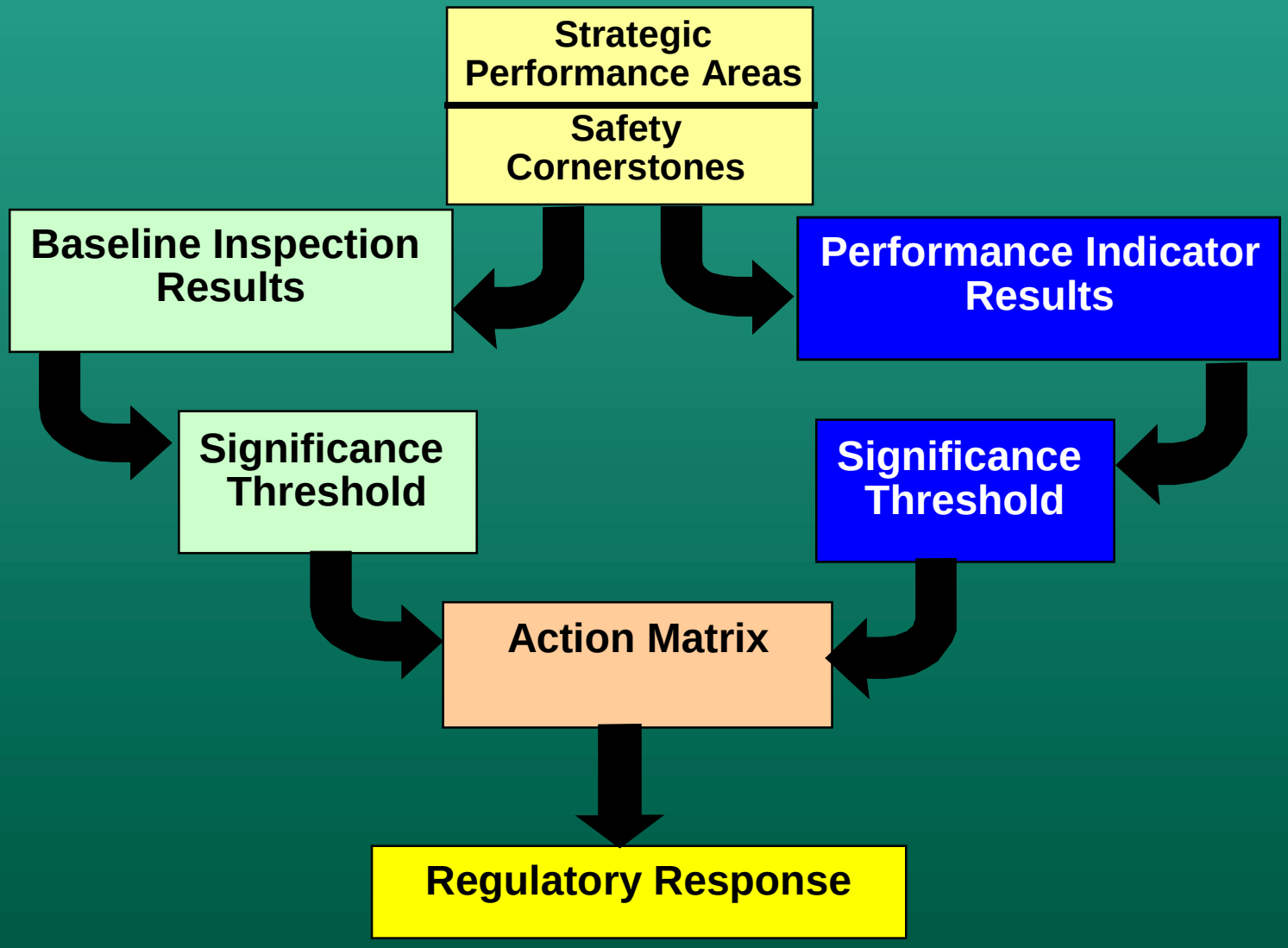
NRC Public Meeting

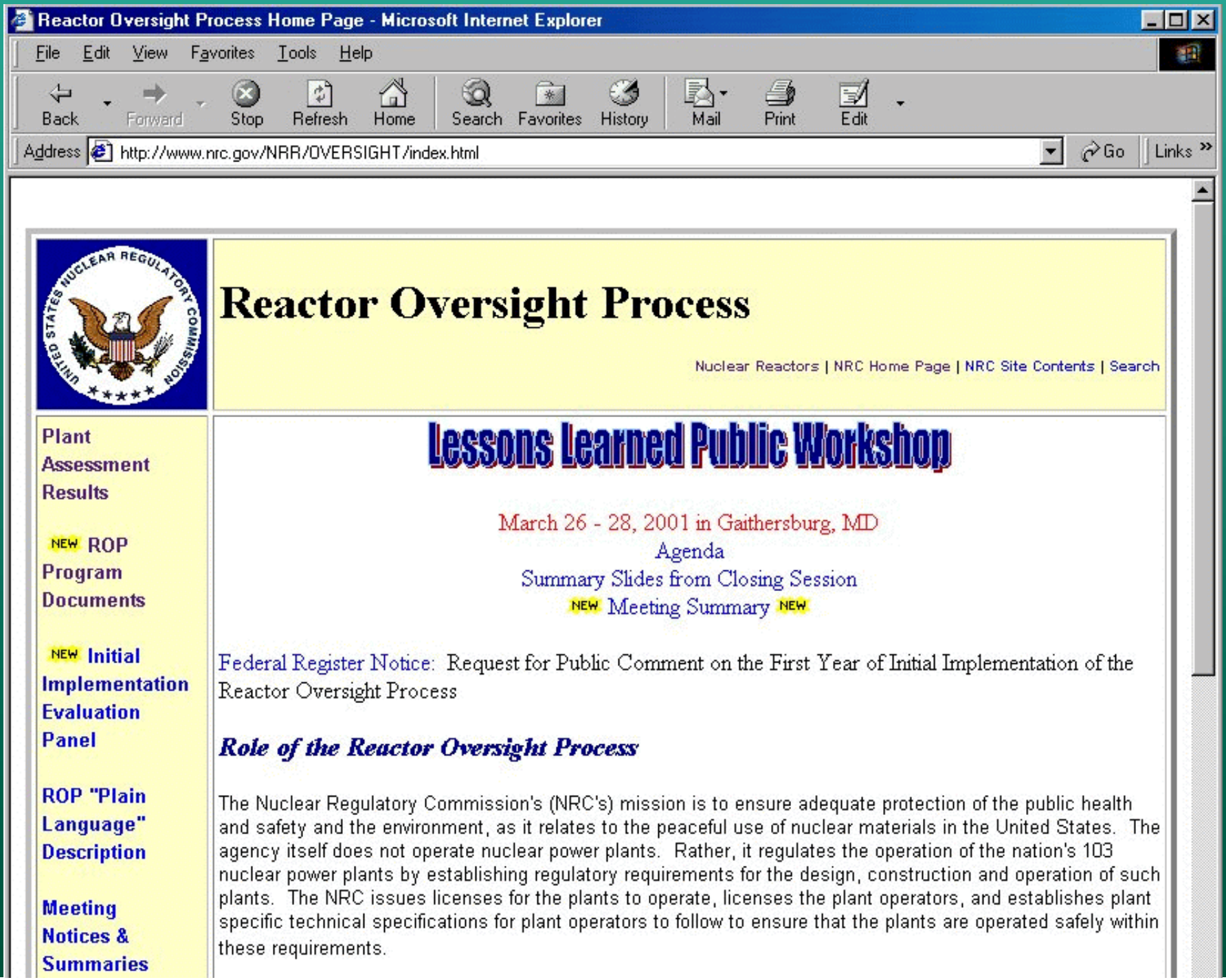
End of Cycle Assessment
Columbia Generating Station
June 21, 2001

Agenda

- Introduction and opening comments - NRC
- Introduction and opening comments - Columbia Generating Station
- Overview of reactor oversight process - NRC
- Presentation of assessment - NRC
- Discussion of assessment - NRC and Columbia Generating Station staff
- Closing remarks - NRC
- Closing remarks - Columbia Generating Station
- Adjourn

Reactor Oversight Process




Reactor Oversight Process Home Page - Microsoft Internet Explorer
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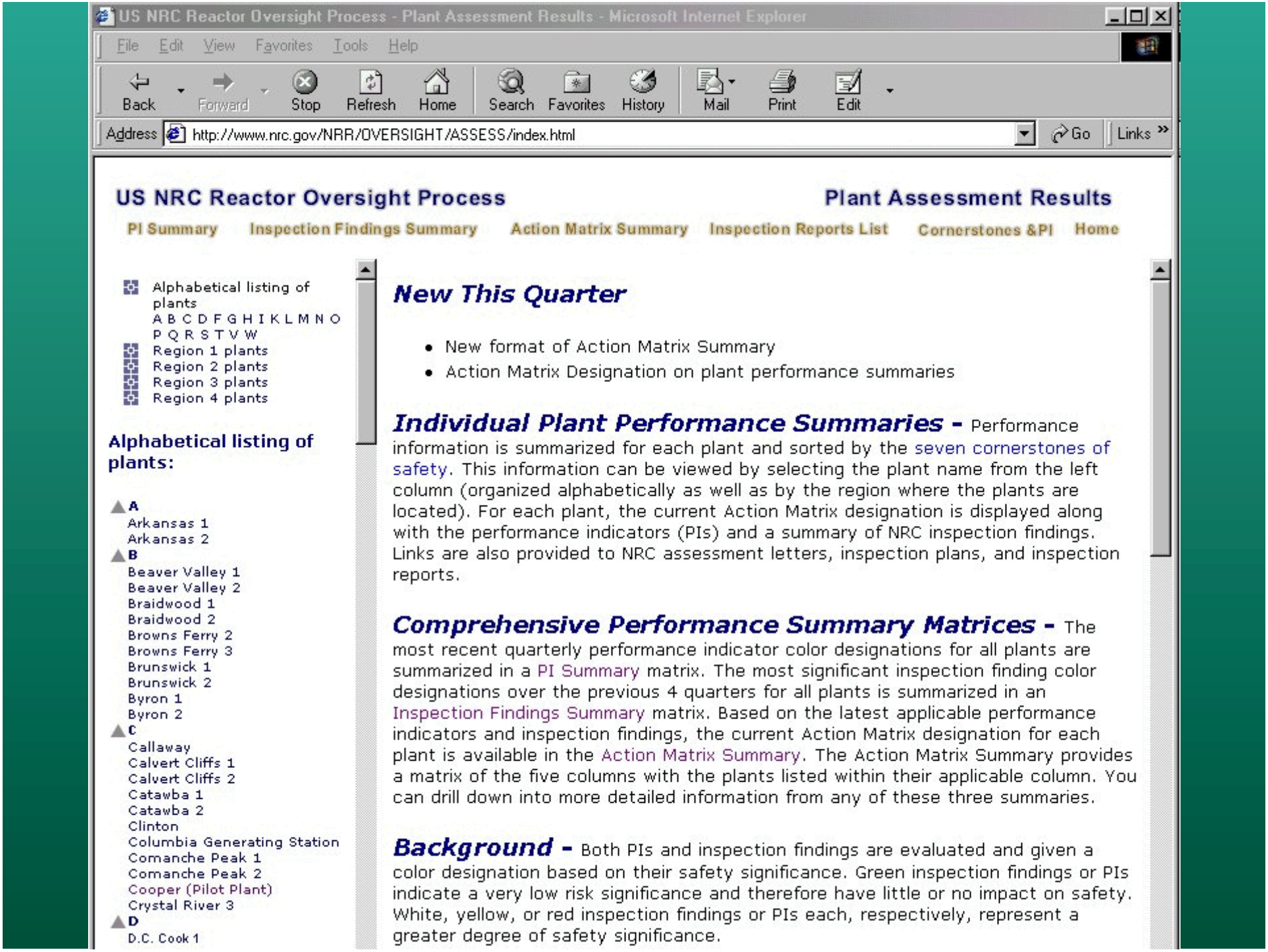
Go Links
Reactor Oversight Process
Nuclear Reactors | NRC Home Page | NRC Site Contents | Search
Plant Assessment Results
NEW ROP Program Documents
NEW Initial Implementation Evaluation Panel
ROP "Plain Language" Description
Meeting Notices & Summaries
Lessons Learned Public Workshop
March 26 - 28, 2001 in Gaithersburg, MD
Agenda
Summary Slides from Closing Session
NEW Meeting Summary NEW
Federal Register Notice: Request for Public Comment on the First Year of Initial Implementation of the Reactor Oversight Process
Role of the Reactor Oversight Process
The Nuclear Regulatory Commission's (NRC's) mission is to ensure adequate protection of the public health and safety and the environment, as it relates to the peaceful use of nuclear materials in the United States. The agency itself does not operate nuclear power plants. Rather, it regulates the operation of the nation's 103 nuclear power plants by establishing regulatory requirements for the design, construction and operation of such plants. The NRC issues licenses for the plants to operate, licenses the plant operators, and establishes plant specific technical specifications for plant operators to follow to ensure that the plants are operated safely within these requirements.


US NRC Reactor Oversight Process - Plant Assessment Results - Microsoft Internet Explorer
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Go Links
US NRC Reactor Oversight Process
Plant Assessment Results
PI Summary Inspection Findings Summary Action Matrix Summary Inspection Reports List Cornerstones &PI Home
Alphabetical listing of plants
A B C D F G H I K L M N O P Q R S T V W
Region 1 plants
Region 2 plants
Region 3 plants
Region 4 plants
Alphabetical listing of plants:
A
Arkansas 1
Arkansas 2
B
Beaver Valley 1
Beaver Valley 2
Braidwood 1
Braidwood 2
Browns Ferry 2
Browns Ferry 3
Brunswick 1
Brunswick 2
Byron 1
Byron 2
C
Callaway
Calvert Cliffs 1
Calvert Cliffs 2
Catawba 1
Catawba 2
Clinton
Columbia Generating Station
Comanche Peak 1
Comanche Peak 2
Cooper (Pilot Plant)
Crystal River 3
D
D.C. Cook 1
New This Quarter
New format of Action Matrix Summary
Action Matrix Designation on plant performance summaries
Individual Plant Performance Summaries - Performance information is summarized for each plant and sorted by the seven cornerstones of safety. This information can be viewed by selecting the plant name from the left column (organized alphabetically as well as by the region where the plants are located). For each plant, the current Action Matrix designation is displayed along with the performance indicators (PIs) and a summary of NRC inspection findings. Links are also provided to NRC assessment letters, inspection plans, and inspection reports.
Comprehensive Performance Summary Matrices - The most recent quarterly performance indicator color designations for all plants are summarized in a PI Summary matrix. The most significant inspection finding color designations over the previous 4 quarters for all plants is summarized in an Inspection Findings Summary matrix. Based on the latest applicable performance indicators and inspection findings, the current Action Matrix designation for each plant is available in the Action Matrix Summary. The Action Matrix Summary provides a matrix of the five columns with the plants listed within their applicable column. You can drill down into more detailed information from any of these three summaries.
Background - Both PIs and inspection findings are evaluated and given a color designation based on their safety significance. Green inspection findings or PIs indicate a very low risk significance and therefore have little or no impact on safety. White, yellow, or red inspection findings or PIs each, respectively, represent a greater degree of safety significance.

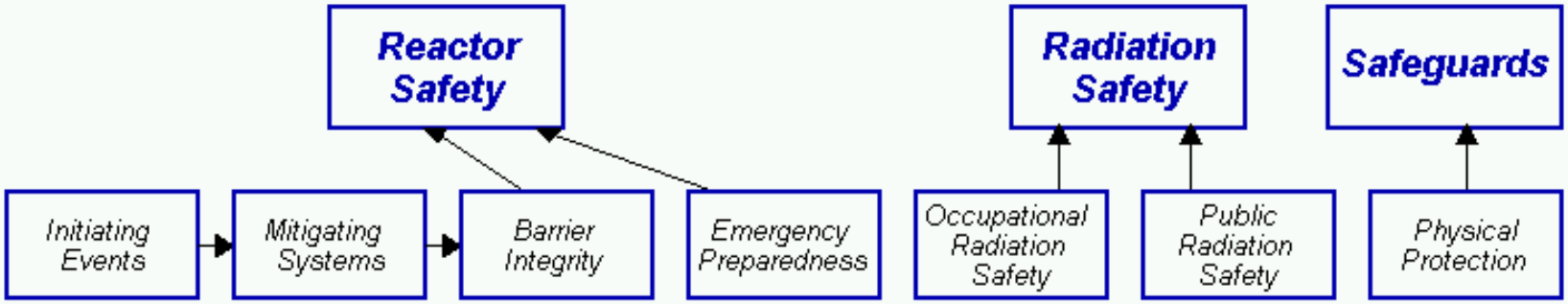
Columbia Generating Station Operating Summary

- Reduced power operation due to economic dispatch, April and May 2000
- Reactor trip due to protective relay control circuit failure (June 2000)
- Planned outage to replace recirculation pump seal, September 2000 (4 days)
- Reactor trip, condenser drainline failure. September 2000
- Otherwise, essentially full power operation

Columbia Generating Station Mid-cycle Assessment

- Completed November 2000
- Licensee Response Column
  - All inspection findings had very low safety significance (Green)
  - No performance indicators required additional NRC oversight

Columbia Generating Station
1Q/2001 Performance Summary



Performance Indicators

Unplanned Scrams (G)	Emergency AC Power System Unavailability (G)	Reactor Coolant System Activity (G)	Drill/Exercise Performance (G)	Occupational Exposure Control Effectiveness (G)	RETS/ODCM Radiological Effluent (G)	Protected Area Equipment (G)
Scrams With Loss of Normal Heat Removal (G)	High Pressure Injection System Unavailability (G)	Reactor Coolant System Leakage (G)	ERO Drill Participation (G)			Personnel Screening Program (G)
Unplanned Power Changes (G)	Heat Removal System Unavailability (G)		Alert and Notification System (G)			FFD/Personnel Reliability Program (G)
	Residual Heat Removal System Unavailability (G)					
	Safety System Functional Failures (G)					

Columbia Generating Station Performance Indicators

- Performance Indicators for the most recent quarter
  - All in the licensee response band
  - No performance indicators required additional NRC oversight

	Initiating Events →	*Mitigating Systems* →	*Barrier Integrity*	*Emergency Preparedness*	*Occupational Radiation Safety*	*Public Radiation Safety*	*Physical Protection*
Most Significant Inspection Findings							
1Q/2001	No findings this quarter	G	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter
4Q/2000	No findings this quarter	G	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter
3Q/2000	No findings this quarter	G	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter
2Q/2000	No findings this quarter	G	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter

Miscellaneous findings

Additional Inspection & Assessment Information

Assessment Reports/Inspection Plans:

- **1Q/2001**
- **4Q/2000**
- **3Q/2000**
- **2Q/2000**

List of Inspection Reports

Columbia Generating Station Inspection Findings

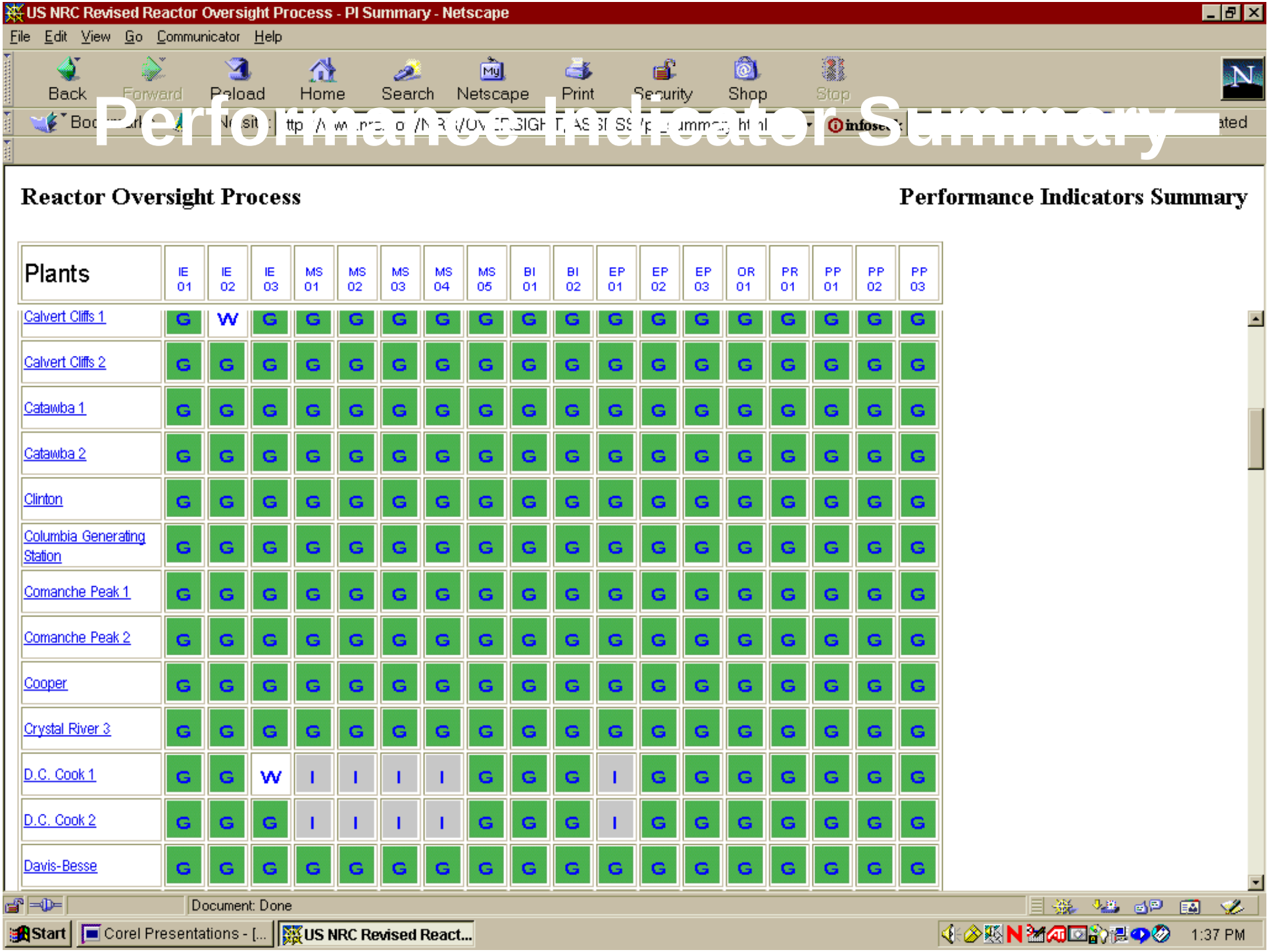
- Findings very low risk significance
- All inspection findings in Columbia Generating Station corrective action system

An Action Matrix is used to assess overall plant safety performance and specify thresholds for NRC Enforcement Actions

		Licensee Response Column	Regulatory Response Column	Degraded Cornerstone Column	Multiple/ Repetitive Degraded Cornerstone Column	Unacceptable Performance Column
RESULTS		All Assessment Inputs (Performance Indicators (PIs) and Inspection Findings) Green; Cornerstone Objectives Fully Met	One or Two White Inputs (in different cornerstones) in a Strategic Performance Area; Cornerstone Objectives Fully Met	One Degraded Cornerstone (2 White Inputs or 1 Yellow Input) or any 3 White Inputs in a Strategic Performance Area; Cornerstone Objectives Met with Minimal Reduction in Safety Margin	Repetitive Degraded Cornerstone, Multiple Degraded Cornerstones, Multiple Yellow Inputs, or 1 Red Input; Cornerstone Objectives Met with Longstanding Issues or Significant Reduction in Safety Margin	Overall Unacceptable Performance; Plants Not Permitted to Operate Within this Band, Unacceptable Margin to Safety
RESPONSE	Regulatory Performance Meeting	None	Branch Chief (BC) or Division Director (DD) Meet with Licensee	DD or Regional Administrator (RA) Meet with Licensee	RA (or EDO) Meet with Senior Licensee Management	Commission meeting with Senior Licensee Management
	Licensee Action	Licensee Corrective Action	Licensee root cause evaluation and corrective action with NRC Oversight	Licensee Self Assessment with NRC Oversight	Licensee Performance Improvement Plan with NRC Oversight	
	NRC Inspection	Risk-Informed Baseline Inspection Program	Baseline and supplemental inspection procedure 95001	Baseline and supplemental inspection procedure 95002	Baseline and supplemental inspection procedure 95003	
	Regulatory Actions	None	Supplemental inspection only	Supplemental inspection only	-10 CFR 2.204 DFI -10 CFR 50.54(f) Letter - CAL/Order	Order to Modify, Suspend, or Revoke Licensed Activities
COMMUNICATION	Assessment Letters	BC or DD review/sign assessment report (w/ inspection plan)	DD review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan) Commission Informed	
	Annual Public Meeting	SRI or BC Meet with Licensee	BC or DD Meet with Licensee	RA (or designee) Discuss Performance with Licensee	EDO (or Commission) Discuss Performance with Senior Licensee Management	Commission Meeting with Senior Licensee Management
	INCREASING SAFETY SIGNIFICANCE ----------->					

Columbia Generating Station Overall Performance

- Preserved public health and safety
- Met all cornerstone objectives
- NRC baseline inspection planned for Columbia Generating Station


US NRC Revised Reactor Oversight Process - PI Summary - Netscape
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Performance Indicator Summary
Reactor Oversight Process
Performance Indicators Summary
Plants
IE 01 IE 02 IE 03 MS 01 MS 02 MS 03 MS 04 MS 05 BI 01 BI 02 EP 01 EP 02 EP 03 OR 01 PR 01 PP 01 PP 02 PP 03
Calvert Cliffs 1 G W G G G G G G G G G G G G G G G G
Calvert Cliffs 2 G G G G G G G G G G G G G G G G G G
Catawba 1 G G G G G G G G G G G G G G G G G G
Catawba 2 G G G G G G G G G G G G G G G G G G
Clinton G G G G G G G G G G G G G G G G G G
Columbia Generating Station G G G G G G G G G G G G G G G G G G
Comanche Peak 1 G G G G G G G G G G G G G G G G G G
Comanche Peak 2 G G G G G G G G G G G G G G G G G G
Cooper G G G G G G G G G G G G G G G G G G
Crystal River 3 G G G G G G G G G G G G G G G G G G
D.C. Cook 1 G G W I I I I G G G I G G G G G G G
D.C. Cook 2 G G G I I I I G G G I G G G G G G G
Davis-Besse G G G G G G G G G G G G G G G G G G
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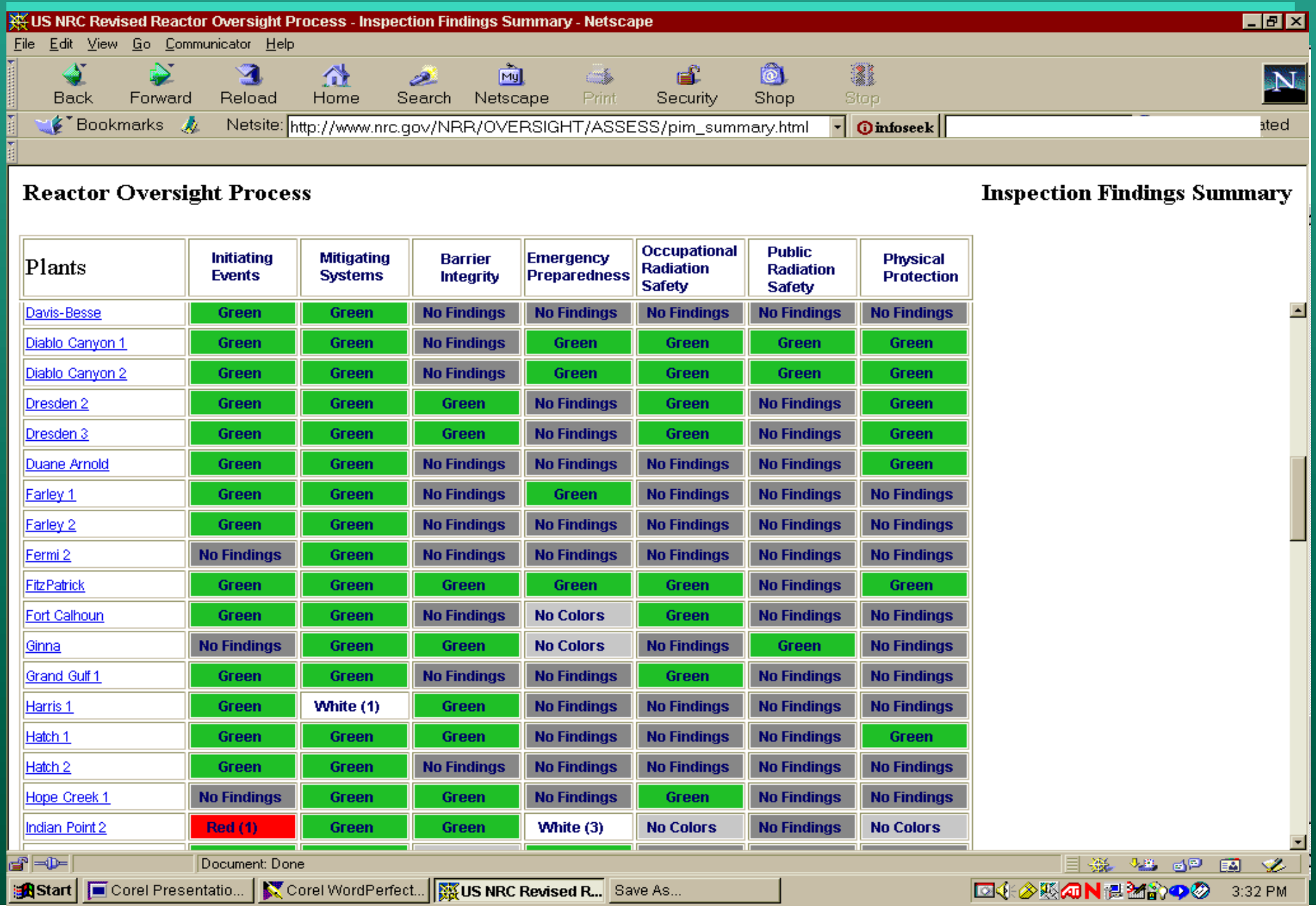
Inspection Finding Summary



Plants	Initiating Events	Mitigating Systems	Barrier Integrity	Emergency Preparedness	Occupational Radiation Safety	Public Radiation Safety	Physical Protection
Davis-Besse	Green	Green	No Findings	No Findings	No Findings	No Findings	No Findings
Diablo Canyon 1	Green	Green	No Findings	Green	Green	Green	Green
Diablo Canyon 2	Green	Green	No Findings	Green	Green	Green	Green
Dresden 2	Green	Green	Green	No Findings	Green	No Findings	Green
Dresden 3	Green	Green	Green	No Findings	Green	No Findings	Green
Duane Arnold	Green	Green	No Findings	No Findings	No Findings	No Findings	Green
Farley 1	Green	Green	No Findings	Green	No Findings	No Findings	No Findings
Farley 2	Green	Green	No Findings	No Findings	No Findings	No Findings	No Findings
Fermi 2	No Findings	Green	No Findings	No Findings	No Findings	No Findings	No Findings
FitzPatrick	Green	Green	Green	Green	Green	No Findings	Green
Fort Calhoun	Green	Green	No Findings	No Colors	Green	No Findings	No Findings
Ginna	No Findings	Green	Green	No Colors	No Findings	Green	No Findings
Grand Gulf 1	Green	Green	No Findings	No Findings	Green	No Findings	No Findings
Harris 1	Green	White (1)	Green	No Findings	No Findings	No Findings	No Findings
Hatch 1	Green	Green	Green	No Findings	No Findings	No Findings	Green
Hatch 2	Green	Green	No Findings	No Findings	No Findings	No Findings	No Findings
Hope Creek 1	No Findings	Green	Green	No Findings	Green	No Findings	No Findings
Indian Point 2	Red (1)	Green	Green	White (3)	No Colors	No Findings	No Colors

Action Matrix Summary

Licensee Response Column	Regulatory Response Column	Degraded Cornerstone Column	Multiple/Repetitive Degraded Cornerstone Column	Unacceptable Performance Column
Arkansas Nuclear 1	Calvert Cliffs 1	Callaway	Indian Point 2	
Arkansas Nuclear 2	Cooper	Kewaunee		
Beaver Valley 1	Fermi 2[1]	Millstone 2		
Beaver Valley 2	FitzPatrick			
Braidwood 1	Harris 1[2]			
Braidwood 2	North Anna 2[3]			
Browns Ferry 2	Oconee 1			
Browns Ferry 3	Point Beach 1			
Brunswick 1	Prairie Island 1[4]			
Brunswick 2	Prairie Island 2[5]			
Byron 1	Quad Cities 1[6]			
Byron 2	Quad Cities 2[7]			
Calvert Cliffs 2	Summer[8]			
Catawba 1	Susquehanna 1[9]			
Catawba 2	Susquehanna 2[10]			
Clinton				
Columbia Generating Station				
Comanche Peak 1				

Major Baseline Inspections through May 31, 2002

- Safety System Design and Performance Capability
  - August 2001
- Problem Identification and Resolution
  - October 2001
- Heat Exchanger Performance
  - December 2001

NRC Public Meeting

- Closing remarks - Columbia Generating Station
- Closing Remarks - NRC
- Adjourn

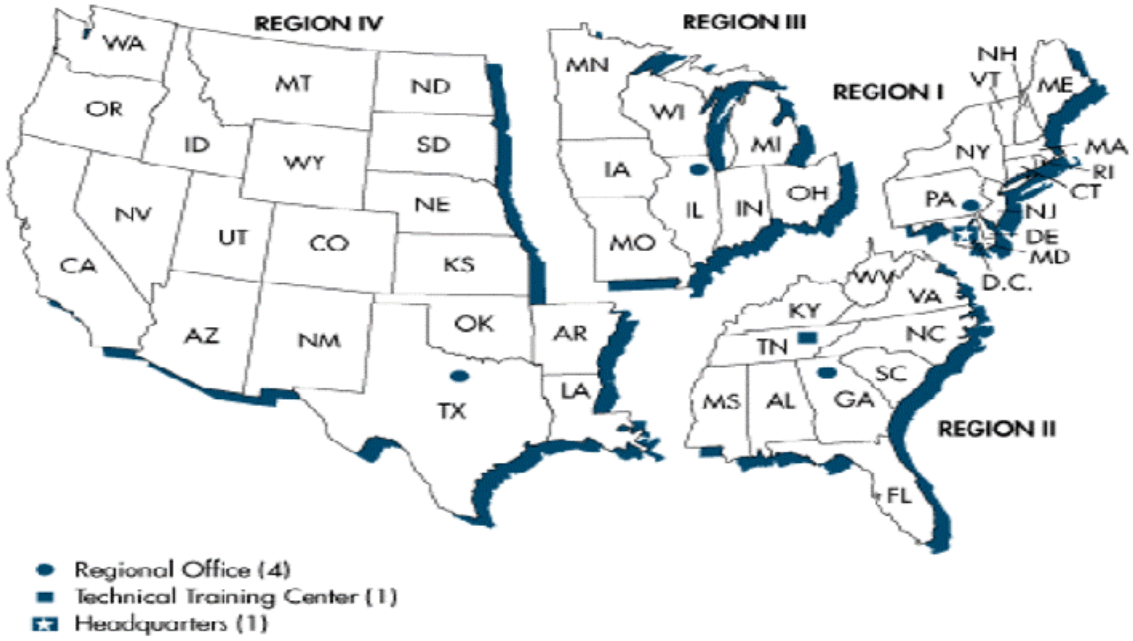
REACTOR OVERSIGHT PROCESS
ANNUAL ASSESSMENT MEETING

Nuclear Regulatory Commission
Region IV

Overview

- Who we are
- Why we are here
- How we inspect and assess plant performance
- Plant performance results
- Questions and Answers

NRC REGIONAL OFFICES

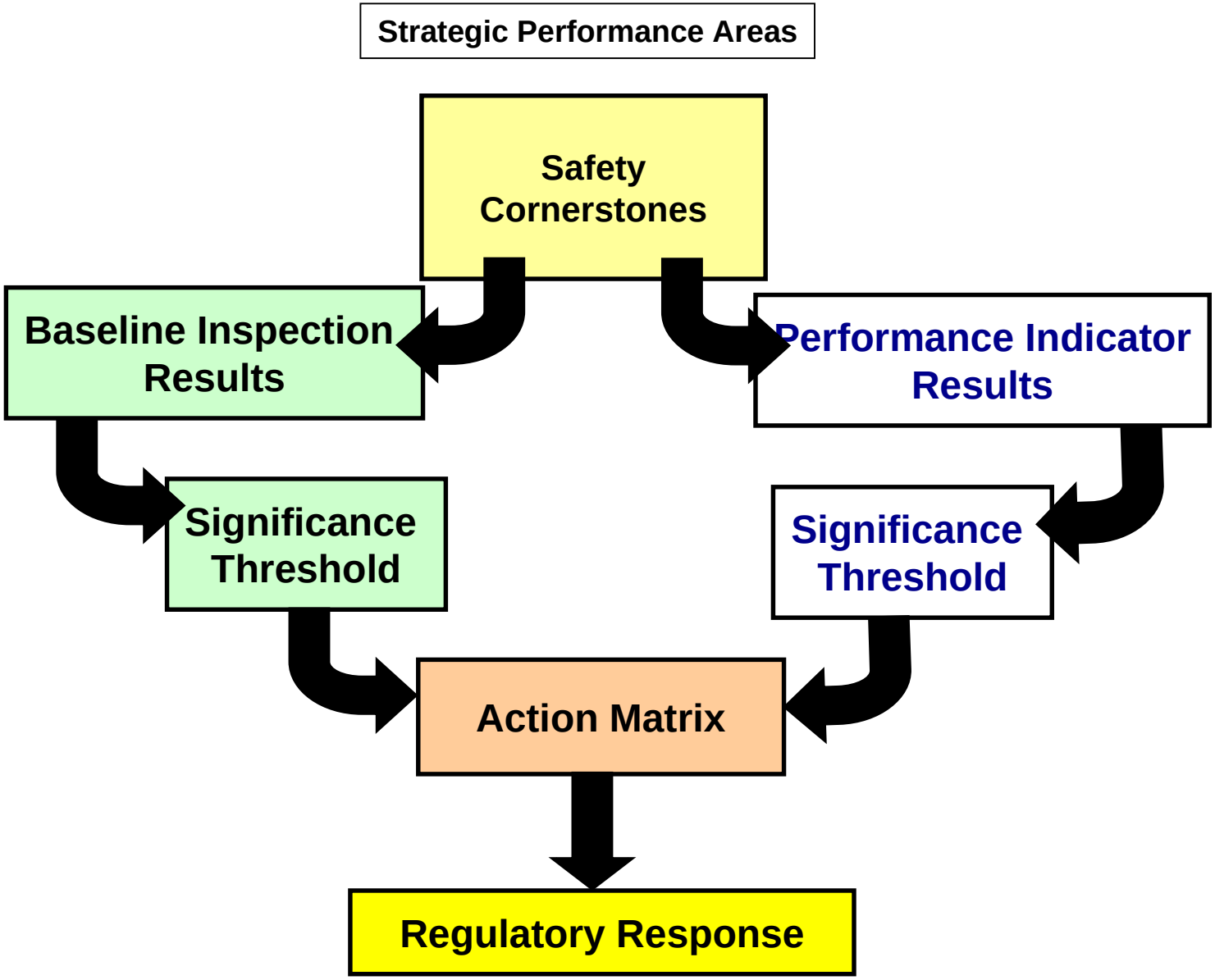


Note: Alaska and Hawaii are included in Region IV.
Source: Nuclear Regulatory Commission

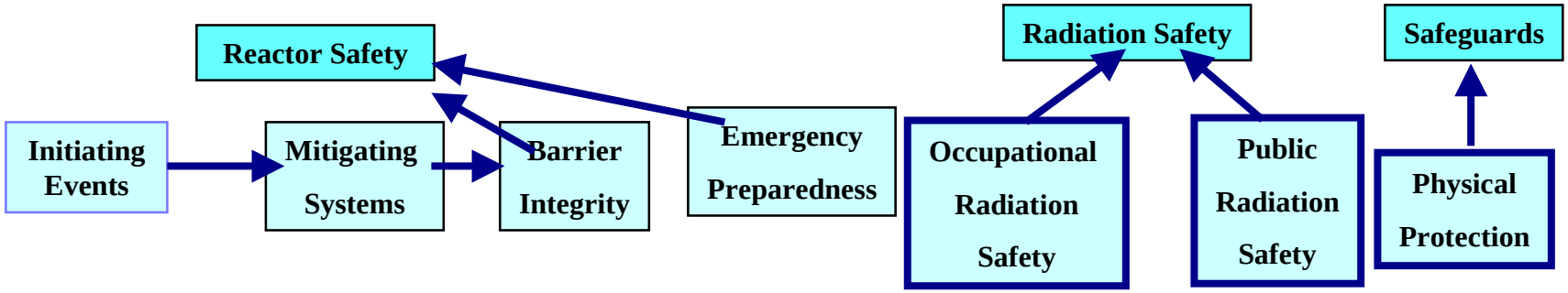
Our Oversight Activities

- Provide assurance plants are operating safely and in accord with the regulations
- Based upon a logical and sound framework
- Uses objective indicators of performance
- Uses inspections focused on key safety areas
- Assessment program triggers regulatory actions

Reactor Oversight Process



The three Strategic Performance Areas are subdivided into seven Cornerstones which are subdivided into 18 Performance Indicators

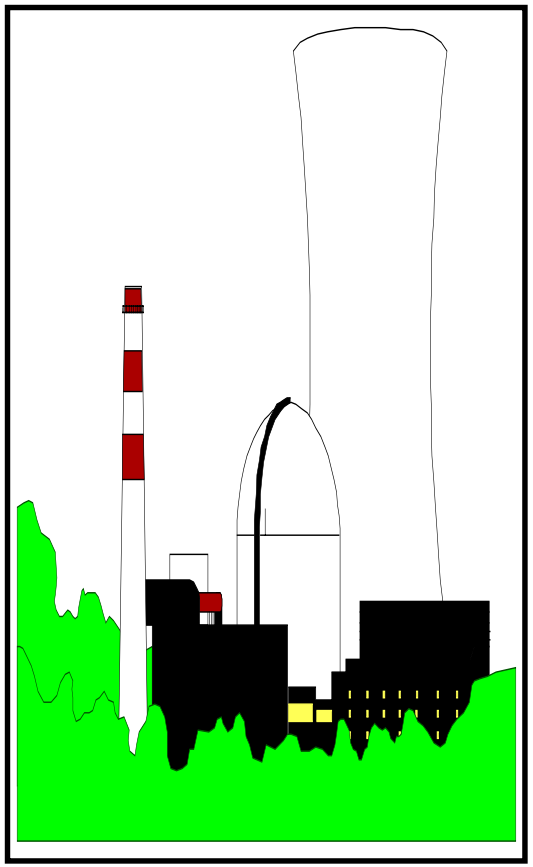


Performance Indicators
Based on data first quarter 2001

- Unplanned Scrams
- Scrams with Loss of Normal Heat Removal
- Unplanned Power Change
- Emergency AC Power System Unavailability
- High Pressure Injection System Unavailability
- Heat Removal System Unavailability
- Residual Heat Removal System Unavailability
- Safety System Functional Failure
- Reactor Coolant System Specific Activity
- Reactor Coolant System Leakage
- Drill/Exercise Performance
- ERO Drill Participation
- Alert and Notification System
- Occupational Exposure Control Effectiveness
- RETS/ODCM Radiological Effluents
- Protected Area Equipment
- Personnel Screening Program
- FFD/Personnel Reliability Program

NRC Conducts Safety Inspections

NRC resident and regional inspectors utilize a Baseline Inspection Program to monitor plant safety performance in each of the Cornerstone of Safety



Key Aspects of Baseline Inspection Program

- Objective evidence of plant safety
- Conducted at all plants
- Emphases safety significant systems, components, activities, and events
- Monitors licensee effectiveness in finding and fixing safety issues
- Inspection reports describe significant findings and non-compliance
- Inspection reports are publicly accessible

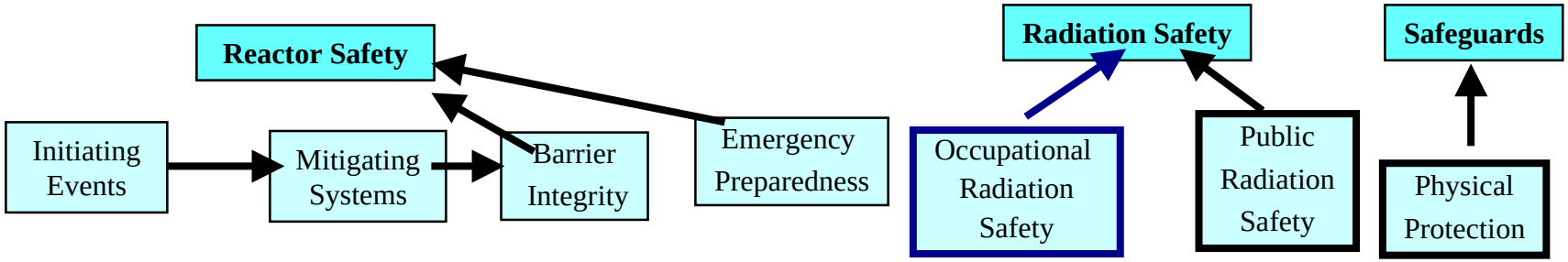
Examples of Baseline Inspections

- Plant safety tours
- Plant control room tours
- Maintenance and alignment of equipment
- Worker radiation protection
- Controls for radiation releases
- Plant security

Event Follow-up and Supplemental Inspection

- Determine causes of performance declines
- Follow-up significant inspection findings
- Review events for significance
- Provides for graduated response

The three Strategic Performance Areas are subdivided into seven Cornerstones which are subdivided into 39 Inspection Procedures



Inspection Procedures

- Adverse Weather
- Evaluation of Changes
- Equipment Alignment
- Fire Protection
- Flood Protection
- Heat Sink Performance
- In-service Inspection
- Operator Requalificaiton
- Maintenance Rulse Implementation
- Maintenance Risk Assessment
- Non-routine Plant Events
- Operability Evaluation
- Operator Workarounds
- Permanent Plant Modifications
- Post Maintenance Testing
- Refueling & Outage
- Safety System Design
- Surveillance Testing
- Temporary Modifications
- Reactor Safety-Emergency Preparedness
- Event Follow-up
- Performance Indicator Verification
- Problem Identification & Resolution
- Exercise Evaluation
- Alert and Notification System
- Emergency Response Organization Augment
- Emergency Action and Plans
- Emergency Preparedness
- Drill Evaluation
- Occupational Radiation Safety
- Access Control
- Radiation Monitoring Instrumentation
- Public Radiation Safety
- Radiation Effluents Treatment
- Radiation Transportation
- Environmental Monitoring
- Security Access Authorization
- Security Search
- Security Response
- Security Plan Change

Key Aspects of Assessment Program

- **Objective assessment of performance**
- **"Action Matrix" to determine agency response to performance:**
  - ***Inspection level increases***
  - ***Management involvement increases***
  - ***Regulatory actions increase***
- **Plant specific assessment letters**
- **Information on NRC public web site**

Colorization Scheme for *Performance Indicators* and *Inspection Findings*

PERFORMANCE INDICATORS

Green: Performance requiring no NRC oversight beyond Baseline Inspection

White: Performance may result in increased NRC oversight

Yellow: Performance that minimally reduces safety margin and requires more NRC oversight

Red: Performance that represents significant reduction in safety, requires more NRC oversight, but provides adequate protection to public health and safety

INSPECTION FINDINGS

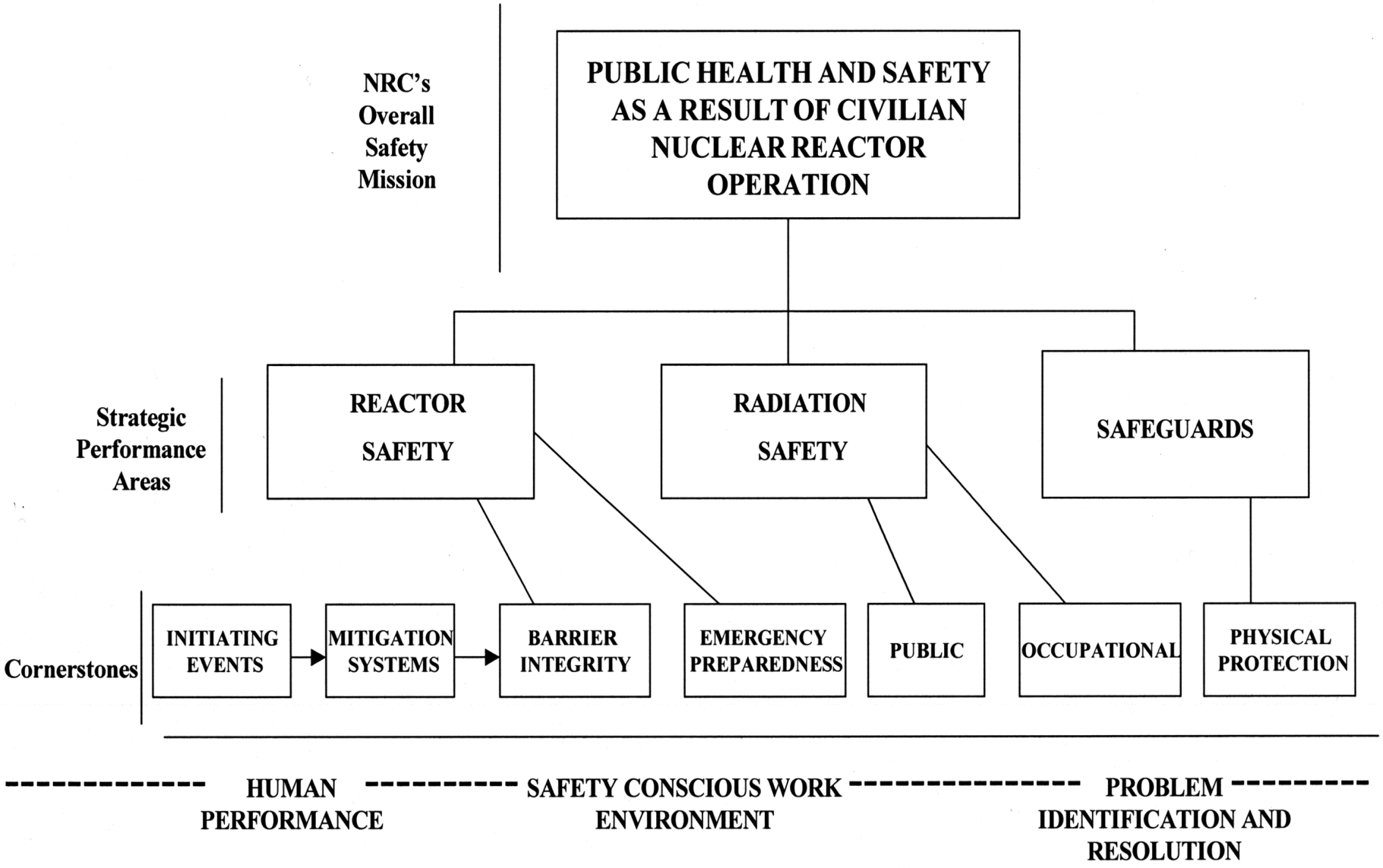
Green: Very low safety issue

White: Low to moderate safety issue

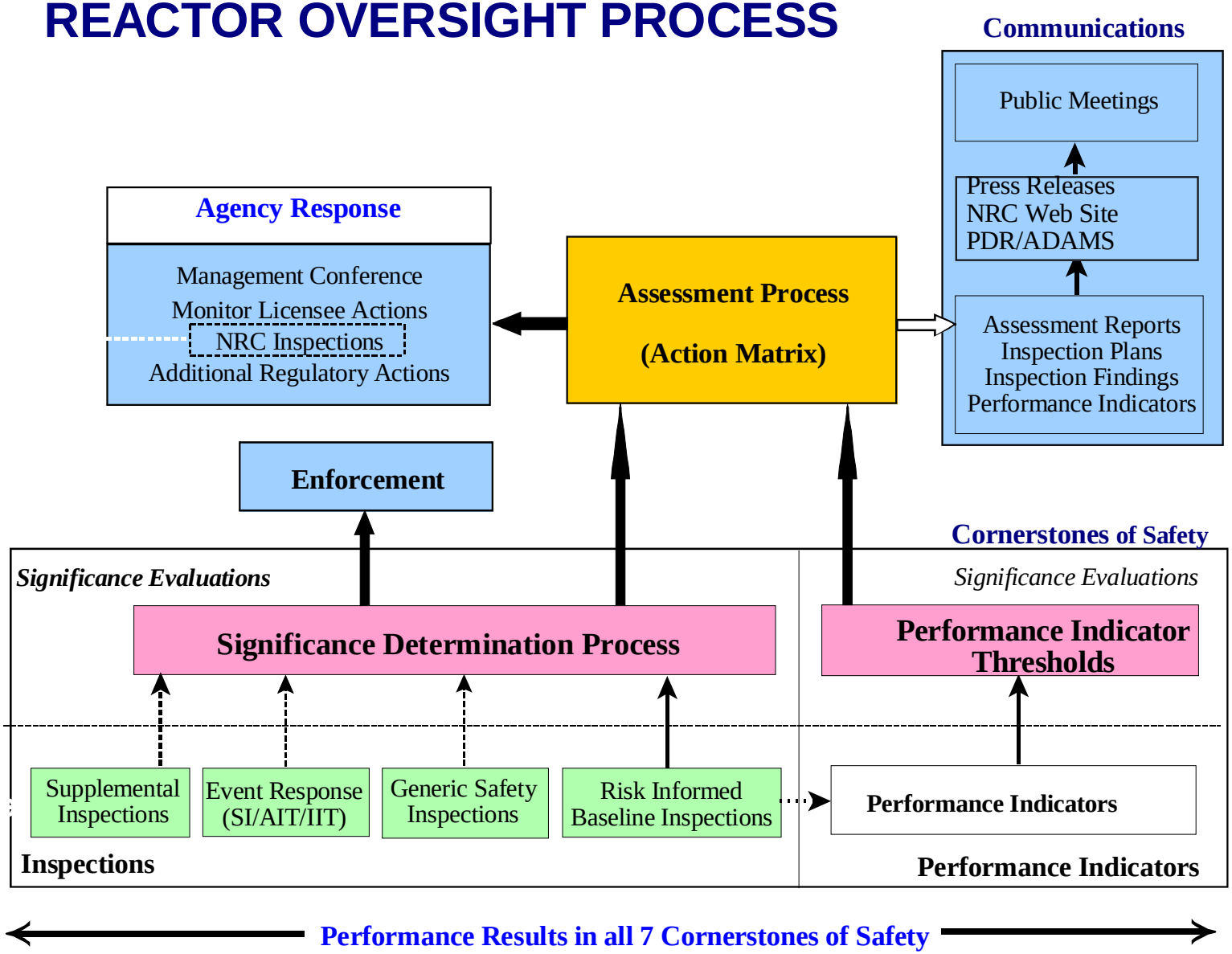
Yellow: Substantial safety issue

Red: High safety issue

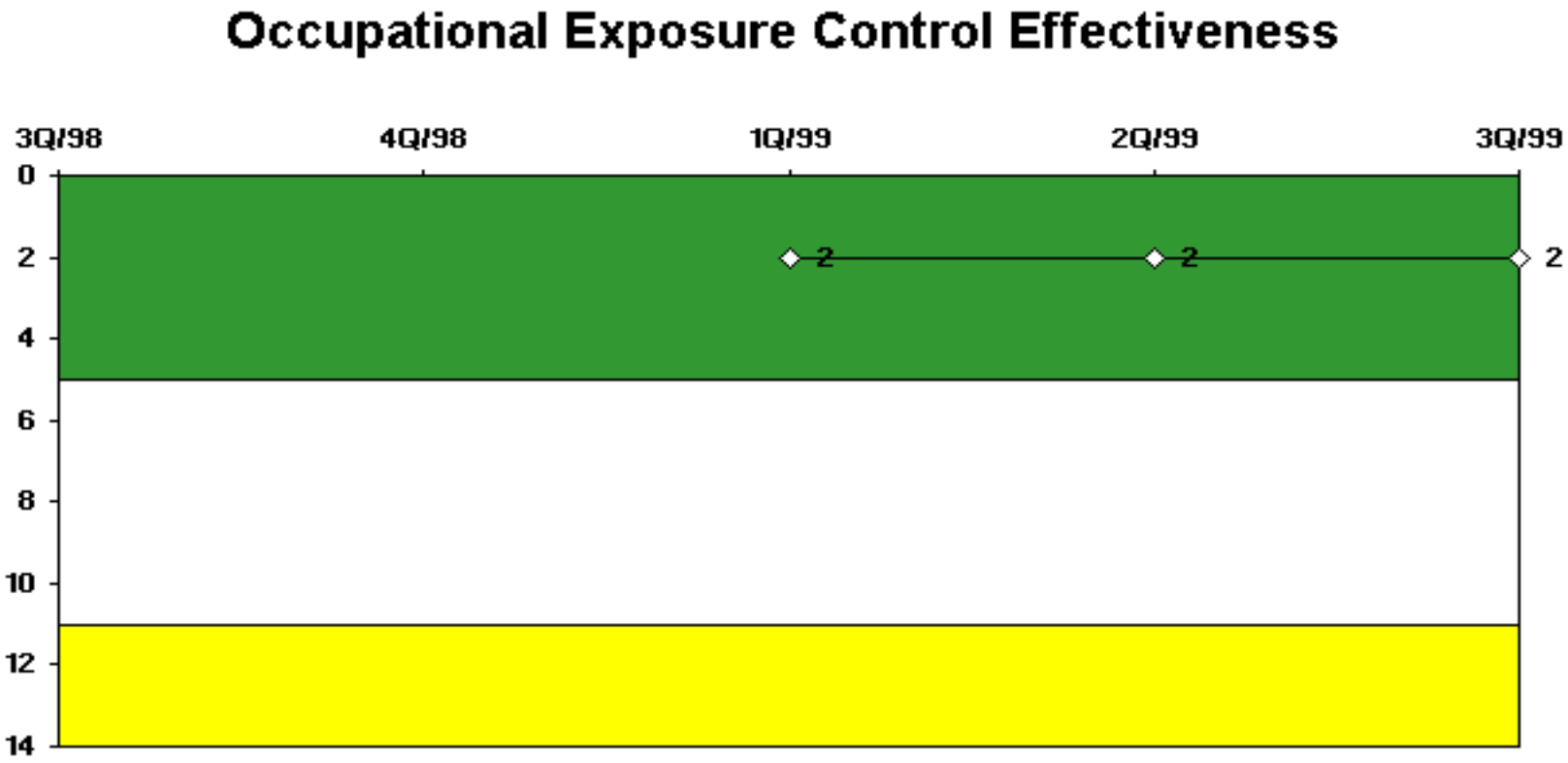
REGULATORY FRAMEWORK



REACTOR OVERSIGHT PROCESS



A Performance Indicator uses objective data to monitor performance in each Cornerstone area



An Action Matrix is used to assess overall plant safety performance and specify thresholds for NRC Enforcement Actions

		Licensee Response Column	Regulatory Response Column	Degraded Cornerstone Column	Multiple/ Repetitive Degraded Cornerstone Column	Unacceptable Performance Column
RESULTS		All Assessment Inputs (Performance Indicators (PIs) and Inspection Findings) Green; Cornerstone Objectives Fully Met	One or Two White Inputs (in different cornerstones) in a Strategic Performance Area; Cornerstone Objectives Fully Met	One Degraded Cornerstone (2 White Inputs or 1 Yellow Input) or any 3 White Inputs in a Strategic Performance Area; Cornerstone Objectives Met with Minimal Reduction in Safety Margin	Repetitive Degraded Cornerstone, Multiple Degraded Cornerstones, Multiple Yellow Inputs, or 1 Red Input; Cornerstone Objectives Met with Longstanding Issues or Significant Reduction in Safety Margin	Overall Unacceptable Performance; Plants Not Permitted to Operate Within this Band, Unacceptable Margin to Safety
RESPONSE	Regulatory Performance Meeting	None	Branch Chief (BC) or Division Director (DD) Meet with Licensee	DD or Regional Administrator (RA) Meet with Licensee	RA (or EDO) Meet with Senior Licensee Management	Commission meeting with Senior Licensee Management
	Licensee Action	Licensee Corrective Action	Licensee root cause evaluation and corrective action with NRC Oversight	Licensee Self Assessment with NRC Oversight	Licensee Performance Improvement Plan with NRC Oversight	
	NRC Inspection	Risk-Informed Baseline Inspection Program	Baseline and supplemental inspection procedure 95001	Baseline and supplemental inspection procedure 95002	Baseline and supplemental inspection procedure 95003	
	Regulatory Actions	None	Supplemental inspection only	Supplemental inspection only	-10 CFR 2.204 DFI -10 CFR 50.54(f) Letter - CAL/Order	Order to Modify, Suspend, or Revoke Licensed Activities
COMMUNICATION	Assessment Letters	BC or DD review/sign assessment report (w/ inspection plan)	DD review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan) Commission Informed	
	Annual Public Meeting	SRI or BC Meet with Licensee	BC or DD Meet with Licensee	RA (or designee) Discuss Performance with Licensee	EDO (or Commission) Discuss Performance with Senior Licensee Management	Commission Meeting with Senior Licensee Management
	INCREASING SAFETY SIGNIFICANCE ----------->					