UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 26, 1998

Mr. Ted C. Feigenbaum
Executive Vice President and
Chief Nuclear Officer
North Atlantic Energy Service Corporation
c/o Mr. Terry L. Harpster
P.O. Box 300
Seabrook, NH 03874

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M97930)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 55 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No 1, in response to your application dated February 12, 1997.

The amendment would modify Technical Specification (TS), Section 6.0, "Administrative Controls," to reflect recent organizational changes and changes to the approval title for the Station Qualified Reviewer Program and to correct an incorrect reference in TS 6.4.3.9.b.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Craig W. Smith, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 55 to NPF-86
2. Safety Evaluation

cc w/encls: See next page

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DATED: March 26, 1998

AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-86 SEABROOK STATION

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL.*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by the North Atlantic Energy Service Corporation, et al. (the licensee), dated February 12, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*North Atlantic Energy Service Corporation (NAESCO) is authorized to act as agent for: North Atlantic Energy Corporation, Canal Electric Company, Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light & Power Department, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, United Illuminating Company, and Vermont Electric Generation and Transmission Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No.55, and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Cecil O. Thomas, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 26, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Overleaf pages have been provided.*

Remove	Insert
6-1	6-1
6-2*	6-2*
6-3*	6-3*
6-4	6-4
6-5*	6-5*
6-6	6-6
6-7	6-7
6-8	6-8
6-8A	6-8A
6-8B	6-8B
6-10	6-10
6-11	6-11
6-12	6-12
6-21	6-21
6-22*	6-22*

6.1 RESPONSIBILITY

6.1.1 The Station Director shall be responsible for overall station operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Manager (or during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Executive Vice President & Chief Nuclear Officer shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions for departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR and updated in accordance with the requirements of 10 CFR 50.71.

b. The Station Director shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

c. The Executive Vice President & Chief Nuclear Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 STATION STAFF

a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;

c. A Health Physics Technician* shall be on site when fuel is in the reactor;

d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation; and

e. Administrative procedures shall be developed and implemented to limit the working hours of station staff who perform safety-related functions, e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel. The amount of overtime worked by station staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

f. The Operations Manager shall have held a Senior Reactor Operator license for Seabrook Station prior to assuming the Operations Manager position.

g. The Assistant Operations Manager shall hold a senior reactor operator license.

*The Health Physics Technician may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

FIGURES 6.2-1 and 6.2-2 are not used.

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION[1]

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3, or 4	MODE 5 or 6
SM[2,4]	1	1
SRO[4]	1	None
RO	2	1
NSO	2	1
STA	1[3]	None

SM - Shift Manager with a Senior Operator license on Unit 1
SRO - Individual with a Senior Operator license on Unit 1
RO - Individual with an Operator license on Unit 1
NSO - Nuclear Systems Operator
STA - Shift Technical Advisor

TABLE NOTATIONS

(1) The shift crew composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewperson being late or absent.

(2) During any absence of the Shift Manager from the control room while the unit is in MODE 1, 2, 3, or 4, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Shift Manager from the control room while the unit is in MODE 5 or 6, an individual with a valid Senior Operator license or Operator license shall be designated to assume the control room command function.

(3) The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Manager or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

(4) While the unit is in MODE 1, 2, 3 or 4, a licensed senior operator, either the SM or SRO, shall be on shift having had at least 6 months of hot operating experience.

6.2.3 INDEPENDENT TECHNICAL REVIEWS

A Technical Review Program shall be established, implemented and maintained to encompass the following Technical Review responsibilities.

FUNCTION

6.2.3.1 The Technical Review Program responsibilities shall encompass:

a. NRC issuances, industry advisories, Licensee Event Reports, and other sources that may indicate areas for improving plant safety;

b. Internal and external operating experience information that may indicate areas for improving plant safety;

c. Plant operating characteristics, plant operations, modifications, maintenance and surveillance to verify independently that these activities are performed safely and correctly and that human errors are reduced as much as practical, and

d. Making detailed recommendations to the Senior Site Official for procedure revisions, equipment modifications or other means of improving nuclear safety and plant reliability.

The Technical Review Program shall utilize several on-site personnel who are independent of the plant management chain to perform the reviews.

RECORDS

6.2.3.2 Written records of technical reviews shall be maintained. As a minimum these records shall include the results of the activities conducted, the status of recommendations made pursuant to Specification 6.2.3.1 and an assessment of company operations related to the reviews performed. A copy of the monthly Technical Review Program report shall be provided to the Senior Site Official.

QUALIFICATIONS

6.2.3.3 Personnel performing reviews pursuant to Technical Specification 6.2.3.1 shall have either a bachelor's degree in engineering or related science and at least 2 years professional level experience, at least 1 year of which shall be in the nuclear field, or equivalent education and experience as defined in ANSI/ANS 3.1, 1981, Section 4.1.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide advisory technical support to the Control Room Commander in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the station.

6.3 TRAINING

6.3.1 A retraining and replacement licensed training program for the station staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and the supplemental requirements specified in NUREG-1021, and shall include familiarization with relevant industry operational experience.

RESPONSIBILITIES

6.4.1.6 The SORC shall be responsible for:

a. Review of: (1) all proposed procedures required by Specification 6.7 and changes thereto, (2) all proposed programs required by Specification 6.7 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Station Director to affect nuclear safety. Procedures and programs required by Specification 6.7 that are designated for review and approval by the Station Qualified Reviewer Program in accordance with Specification 6.4.2 do not require SORC review.

b. Review of all proposed tests and experiments that affect nuclear safety;

c. Review of all proposed changes to Appendix "A" Technical Specifications;

d. Review of all proposed changes or modifications to station systems or equipment that affect nuclear safety;

e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Executive Vice President & Chief Nuclear officer and to the Nuclear Safety Audit Review Committee (NSARC);

f. Review of all REPORTABLE EVENTS;

g. Review of station operations to detect potential hazards to nuclear safety;

h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Station Director or the NSARC;

i. Not used;

j. Not used;

k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Executive Vice President & Chief Nuclear Officer and to the NSARC;

l. Review of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment System; and

m. Review of the Fire Protection Program and implementing instructions and submittal of recommended Fire Protection Program changes to the NSARC.

6.4.1.7 The SORC shall:

a. Recommend in writing to the Station Director approval or disapproval of items considered under Specification 6.4.1.6a. through d;

b. Render determinations in writing with regard to whether or not each item considered under Specification 6.4.1.6a. through e. constitutes an unreviewed safety question; and

c. Provide written notification within 24 hours to the Executive Vice President & Chief Nuclear Officer and the NSARC of disagreement between the SORC and the Station Director however, the Station Director shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.4.1.8 The SORC shall maintain written minutes of each SORC meeting that, at a minimum, document the results of all SORC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President & Chief Nuclear Officer and the NSARC.

6.4.2 STATION QUALIFIED REVIEWER PROGRAM

FUNCTION

6.4.2.1 The Station Director may establish a Station Qualified Reviewer Program whereby required reviews of designated procedures or classes of procedures required by Specification 6.4.1.6.a are performed by Station Qualified Reviewers and approved by the designated department heads. These reviews are in lieu of reviews by the SORC. However, procedures which require a 10 CFR 50.59 evaluation must be reviewed by the SORC.

RESPONSIBILITIES

6.4.2.2 The Station Qualified Reviewer Program shall:

a. Provide for the review of designated procedures, programs, and changes thereto by a Qualified Reviewer(s) other than the individual who prepared the procedure, program, or change.

b. Provide for cross-disciplinary review of procedures, programs, and changes thereto when organizations other than the preparing organization are affected by the procedure, program, or change.

c. Ensure cross-disciplinary reviews are performed by a Qualified Reviewer(s) in affected disciplines, or by other persons designated by cognizant department heads as having specific expertise required to assess a particular procedure, program or change. Cross-disciplinary reviewers may function as a committee.

d. Provide for a screening of designated procedures, programs and changes thereto to determine if an evaluation should be performed in accordance with the provisions of 10 CFR 50.59 to verify that an unreviewed safety question does not exist. This screening will be performed by personnel trained and qualified in performing 10 CFR 50.59 evaluations.

e. Provide for written recommendation by the Qualified Reviewer(s) to the responsible department head for approval or disapproval of procedures and programs considered under Specification 6.4.1.6a and that the procedure or program was screened by a qualified individual and found not to require a 10 CFR 50.59 evaluation.

6.4.2.3 If the responsible department head determines that a new program, procedure, or change thereto requires a 10 CFR 50.59 evaluation, that designated department head will ensure the required evaluation is performed to determine if the new procedure, program, or change involves an unreviewed safety question. The new procedure, program, or change will then be forwarded with the 10 CFR 50.59 evaluation to SORC for review.

6.4.2.4 Personnel recommended to be Station Qualified Reviewers shall be designated in writing by the Station Director for each procedure, program, or class of procedure or program within the scope of the Station Qualified Reviewer Program.

6.4.2.5 Temporary procedure changes shall be made in accordance with Specification 6.7.3 with the exception that changes to procedures for which reviews are assigned to Qualified Reviewers will be reviewed and approved as described in Specification 6.4.2.2.

RECORDS

6.4.2.6 The review of procedures and programs performed under the Station Qualified Reviewer Program shall be documented in accordance with administrative procedures.

TRAINING AND QUALIFICATION

6.4.2.7 The training and qualification requirements of personnel designated as a Qualified Reviewer in accordance with the Station Qualified Reviewer Program shall be in accordance with administrative procedures. Qualified reviewers shall have:

a. A Bachelors degree in engineering, related science, or technical discipline, and two years of nuclear power plant experience;

OR

b. Six years of nuclear power plant experience;

OR

c. An equivalent combination of education and experience as approved by the designated department head.

6.4.3 NUCLEAR SAFETY AUDIT REVIEW COMMITTEE (NSARC)

FUNCTION

6.4.3.1 The NSARC shall function to provide independent review and audit of designated activities. The NSARC shall report to and advise the Executive Vice President & Chief Nuclear Officer on those areas of responsibility specified in Specifications 6.4.3.7 and 6.4.3.8.

COMPOSITION

6.4.3.2 The NSARC shall be composed of at least five (5) individuals. The Chairman, Vice Chairman and members, including designated alternates, shall be appointed in writing by the Executive Vice President & Chief Nuclear Officer. Collectively, the individuals appointed to the NSARC should have experience and expertise in the following areas:

a. Nuclear power plant operations.
b. Nuclear engineering.
c. Chemistry and radiochemistry.
d. Metallurgy.
e. Instrumentation and control.
f. Radiological safety.
g. Mechanical and electrical engineering, and
h. Quality assurance practices.

Each member shall meet the qualifications of ANSI 3.1-1978, Section 4.7.

ALTERNATES

6.4.3.3 All alternate members shall be appointed in writing by the Executive Vice President & Chief Nuclear Officer to serve on a temporary basis; however, no more than a minority shall participate as voting members in NSARC activities at any one time.

CONSULTANTS

6.4.3.4 Consultants shall be utilized as determined by the NSARC to provide expert advice to the NSARC.

MEETING FREQUENCY

6.4.3.5 The NSARC shall meet at least once per 6 months ± 6 weeks.

QUORUM

6.4.3.6 The quorum of the NSARC necessary for the performance of the NSARC review and audit functions of these Technical Specifications shall consist of the Chairman or Vice-Chairman and at least four NSARC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit. The Vice Chairman, or his designated alternate, can participate as an NSARC member when the Chairman is in attendance.

AUDITS

6.4.3.8 Audits of station activities shall be performed under the cognizance of the NSARC. The audits shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the specified interval provided the combined time interval for any three consecutive intervals shall not exceed 3.25 times the specified interval. These audits shall encompass:

a. The conformance of station operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;

b. The performance, training, and qualifications of the entire station staff at least once per 12 months;

c. The results of actions taken to correct deficiencies occurring in station equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;

e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;

f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;

g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;

h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;

i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;

j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;

k. Not used;

l. Not used; and

m. Any other area of station operation considered appropriate by the NSARC or the Executive Vice President & Chief Nuclear Officer.

RECORDS

6.4.3.9 Records of NSARC activities shall be prepared and distributed as indicated below:

a. Minutes of each NSARC meeting shall be prepared and forwarded to Executive Vice President & Chief Nuclear Officer within 30 working days following each meeting;

b. Reports of reviews encompassed by Specification 6.4.3.7 shall be included in the minutes where applicable or forwarded under separate cover to the Executive Vice President & Chief Nuclear Officer within 30 working days following completion of the review; and

c. Audit reports encompassed by Specification 6.4.2.8 shall be forwarded to the Executive Vice President & Chief Nuclear Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.5 REPORTABLE EVENT ACTION

The following actions shall be taken for REPORTABLE EVENTS:

a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and

b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review shall be submitted to the NSARC and the Executive Vice President & Chief Nuclear Officer.

6.6 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Executive Vice President & Chief Nuclear Officer and the NSARC shall be notified within 24 hours;

b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;

c. The Safety Limit Violation Report shall be submitted to the Commission, the NSARC, and the Executive Vice President & Chief Nuclear Officer within 14 days of the violation; and

d. Operation of the station shall not be resumed until authorized by the Commission.

6.7 PROCEDURES AND PROGRAMS

6.7.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;

b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;

c. Not used;

d. Not used;

e. PROCESS CONTROL PROGRAM implementation;

f. OFFSITE DOSE CALCULATION MANUAL implementation;

g. Quality Assurance Program for effluent and environmental monitoring;

h. Fire Protection Program implementation; and

i. Technical Specification Improvement Program implementation.

6.7.2 The Station Director may designate specific procedures and programs or classes of procedures and programs to be reviewed in accordance with the Station Qualified Reviewer Program in lieu of review by the SORC. The review per the Qualified Reviewer Program shall be in accordance with Specification 6.4.2.

6.7.3 Procedures and programs listed in Specification 6.7.1, and changes thereto, shall be approved by the Station Director or by cognizant department head or Directors who are designated as the Approval Authority by the Station Director, as specified in administrative procedures. The Approval Authority for each procedure and program or class of procedure and program shall be specified in administrative procedures.

6.7.4 Each procedure of Specification 6.7.1, and changes thereto, shall be reviewed by the SORC and shall be approved by the Station Director, or be reviewed and approved in accordance with the Station Qualified Reviewer Program, prior to implementation. Each procedure of Specification 6.7.1 shall be reviewed periodically as set forth in administrative procedures.

6.7.5 Changes to procedures of Specification 6.7.1 may be made prior to SORC review provided:

a. The intent of the original procedure is not altered;

b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license; and

c. The change is documented, reviewed by the SORC and approved by the Station Director, or reviewed and approved in accordance with the Station Qualified Reviewer Program, within 14 days of implementation.

HIGH RADIATION AREA

6.11.1 (Continued)

radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device that continuously indicates the radiation dose rate in the area; or

b. A radiation monitoring device that continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or

c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified in the Radiation Work Permit.

6.11.2 In addition to the requirements of Specification 6.11.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface that the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Manager on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP that shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.12 PROCESS CONTROL PROGRAM (PCP)

6.12.1 The PCP shall be approved by the Commission prior to implementation.

6.12.2 Licensee-initiated changes to the PCP:

a. Shall be submitted to the Commission in the Annual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:

PROCESS CONTROL PROGRAM (PCP)

6.12.2 (Continued)

1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;

2) A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and

3) Documentation of the fact that the change has been reviewed and found acceptable by the SORC.

b. Shall become effective upon review and acceptance by the SORC.

6.13 OFFSITE DOSE CALCULATION MANUAL (ODCM)

6.13.1 The OCDM shall be approved by the Commission prior to implementation.

6.13.2 Licensee-initiated changes to the ODCM:

a. Changes to Part A shall be submitted to and approved by the NRC staff prior to implementation.

b. Changes to Part B shall be submitted to the Commission in the Annual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:

1) Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered, dated and containing the revision number, together with appropriate analyses or evaluations justifying the change(s);

2) A determination that the change will not reduce the accuracy or reliability of dose calculations or Setpoint determinations; and

3) Documentation of the fact that the change has been reviewed and found acceptable by the SORC.

c. Changes to Part B shall become effective upon review and acceptance by the SORC.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-86

NORTH ATLANTIC ENERGY SERVICE CORPORATION

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

1.0 INTRODUCTION

By letter dated February 12, 1997, the North Atlantic Energy Service Corporation (the licensee) submitted a request for changes to the Seabrook Station, Technical Specifications (TSs). The requested changes would modify TS, Section 6.0, "Administrative Controls" to reflect recent organizational changes and changes to the approval title for the Station Qualified Reviewer Program and to correct an incorrect reference in TS 6.4.3.9.b.

2.0 BACKGROUND

The amendment request was submitted in response to organizational changes implemented by Northeast Utilities (NU) on October 1, 1996. The NU Nuclear Organization changes were reported to the U. S. Nuclear Regulatory Commission (NRC) in a letter dated October 3, 1996. Additional organizational changes were announced in January 1997 and became effective on February 1, 1997. The license amendment would modify TS, Section 6.0 to reflect these organizational changes. The license amendment would also modify TS, Section 6.0 to reflect title changes for the positions of Station Director, Shift Manager, Nuclear System Operator, and crew person.

In addition, the license amendment would modify TS 6.4.2, Station Qualified Reviewer Program, to use the term "department head" in lieu of "managers" as the approval authority for procedures reviewed under the Station Qualified Reviewer Program.

The incorrect reference in TS 6.4.3.9.b was inadvertently ommitted from a previous license amendment.

3.0 EVALUATION

The organizational changes proposed by the licensee are administrative in nature. The proposed license amendment would change the title of the Senior Vice President to Executive Vice President and Chief Nuclear Officer. The position of Executive Director - Nuclear Production would be eliminated. The duties previously held by the Executive Director - Nuclear Production would be assumed by the Executive Vice President and Chief Nuclear Officer. Other organizational changes would include title changes for the Station Manager to Station Director, Shift Superintendent to Shift Manager, Auxiliary Operator to Nuclear Systems Operator, and crewman to crewperson. The proposed organizational changes are administrative in nature in

that they do not eliminate any of the duties or responsibilities listed in the TSs; they only modify the title of the individual listed in the TSs assigned to carry out those responsibilities. The proposed organizational changes will have no effect on the safe operation of the facility.

The proposed change to the Station Qualified Reviewer Program is an administrative change. The change would reassign approval responsibilities for procedures reviewed in accordance with the Station Qualified Reviewer Program to the correct departmental position. The change replaces the term "manager" with the term "department head" or "designated department head" with respect to the Station Qualified Reviewer Program. The Station Director must still designate the specific procedures and programs, or classes of procedures and programs to be reviewed in accordance with the Station Qualified Reviewer Program and must still designate the approval authority for those procedures and programs. The proposed changes to the Station Qualified Reviewer Program will have not effect on the safe operation of the facility.

The proposed change would correct an incorrect reference in TS 6.4.3.9.b. TS 6.4.3.9.b references TS 6.4.2.7 but should reference TS 6.4.3.7 as the TS controlling reports and reviews of the Nuclear Safety Audit Review Committee. This reference correction was inadvertently omitted from a previous license amendment.

In consideration of the foregoing, the changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Craig W. Smith

Date: March 26, 1998