

May 19, 1995

Mr. Ted C. Feigenbaum  
Senior Vice President  
and Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
Post Office Box 300  
Seabrook, NH 03874

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SUBJECT: AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NPF-86:  
ADMINISTRATIVE CONTROLS - LICENSE AMENDMENT REQUEST 94-03  
(TAC M90116)

Dear Mr. Feigenbaum:

The Commission has issued the enclosed Amendment No. 39 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No. 1, in response to your application dated October 7, 1994.

The amendment deletes from the Appendix A Technical Specifications certain audit responsibilities of the Nuclear Safety Audit Review Committee and certain review responsibilities of the Station Operation Review Committee relating to the Emergency Plan and the Security Plan and their respective implementing procedures. The proposed changes are consistent with the guidance of Generic Letter 93-07.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,  
Original signed by

Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-443  
Serial No. SEA-95-011

Enclosures: 1. Amendment No. 39 to NPF-86  
2. Safety Evaluation

cc w/enclosures: See next page

DOCUMENT NAME: G:\DEAGAZIO\90116AMD

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 19, 1995

Mr. Ted C. Feigenbaum  
Senior Vice President  
and Chief Nuclear Officer  
North Atlantic Energy Service Corporation  
Post Office Box 300  
Seabrook, NH 03874

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Sincerely,

A handwritten signature in cursive script, reading "Albert W. De Agazio".

Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-443  
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cc w/enclosures: See next page

Mr. Ted C. Feigenbaum  
North Atlantic Energy Service Corporation

Seabrook Station, Unit No. 1

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTH ATLANTIC ENERGY SERVICE CORPORATION, ET AL\*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39  
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by North Atlantic Energy Service Corporation, et al. (the licensee), dated October 7, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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\*North Atlantic Energy Service Company (NAESCO) is authorized to act as agent for the: North Atlantic Energy Corporation, Canal Electric Company, The Connecticut Light and Power Company, Great Bay Power Corporation, Hudson Light and Power Department, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, New Hampshire Electric Cooperative, Inc., Taunton Municipal Light Plant, and The United Illuminating Company, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 39, and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. NAESCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 19, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of Appendix A, Technical Specifications, with the attached pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf pages have been provided.

Remove

6-7

6-8\*

6-9\*

6-10

6-11\*

6-12

Insert

6-7

6-8\*

6-9\*

6-10

6-11\*

6-12

RESPONSIBILITIES

6.4.1.6 The SORC shall be responsible for:

- a. Review of: (1) all proposed procedures required by Specification 6.7 and changes thereto, (2) all proposed programs required by Specification 6.7 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety. Procedures and programs required by Specification 6.7 that are designated for review and approval by the Station Qualified Reviewer Program in accordance with Specification 6.4.2 do not require SORC review.
- b. Review of all proposed tests and experiments that affect nuclear safety;
- c. Review of all proposed changes to Appendix "A" Technical Specifications;
- d. Review of all proposed changes or modifications to station systems or equipment that affect nuclear safety;
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Executive Director - Nuclear Production and to the Nuclear Safety Audit Review Committee (NSARC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of station operations to detect potential hazards to nuclear safety;
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Station Manager or the NSARC;
- i. Not used;
- j. Not used;
- k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Executive Director - Nuclear Production and to the NSARC;
- l. Review of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment System; and
- m. Review of the Fire Protection Program and implementing instructions and submittal of recommended Fire Protection Program changes to the NSARC.

## ADMINISTRATIVE CONTROLS

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### 6.4.1.7 The SORC shall:

- a. Recommend in writing to the Station Manager approval or disapproval of items considered under Specification 6.4.1.6a. through d;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.4.1.6a. through e. constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Executive Director - Nuclear Production and the NSARC of disagreement between the SORC and the Station Manager however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

## RECORDS

6.4.1.8 The SORC shall maintain written minutes of each SORC meeting that, at a minimum, document the results of all SORC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Executive Director-Nuclear Production and the NSARC.

## 6.4.2 STATION QUALIFIED REVIEWER PROGRAM

### FUNCTION

6.4.2.1 The Station Manager may establish a Station Qualified Reviewer Program whereby required reviews of designated procedures or classes of procedures required by Specification 6.4.1.6.a are performed by Station Qualified Reviewers and approved by designated managers. These reviews are in lieu of reviews by the SORC. However, procedures which require a 10 CFR 50.59 evaluation must be reviewed by the SORC.

### RESPONSIBILITIES

#### 6.4.2.2 The Station Qualified Reviewer Program shall:

- a. Provide for the review of designated procedures, programs, and changes thereto by a Qualified Reviewer(s) other than the individual who prepared the procedure, program, or change.
- b. Provide for cross-disciplinary review of procedures, programs, and changes thereto when organizations other than the preparing organization are affected by the procedure, program, or change.
- c. Ensure cross-disciplinary reviews are performed by a Qualified Reviewer(s) in affected disciplines, or by other persons designated by cognizant Managers or Directors as having specific expertise required to assess a particular procedure, program or change. Cross-disciplinary reviewers may function as a committee.

REVIEW

6.4.3.7 The NSARC shall be responsible for the review of:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems that involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments that involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of station equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the SORC.

AUDITS

6.4.3.8 Audits of station activities shall be performed under the cognizance of the NSARC. The audits shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the specified interval provided the combined time interval for any three consecutive intervals shall not exceed 3.25 times the specified interval. These audits shall encompass:

- a. The conformance of station operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training, and qualifications of the entire station staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in station equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;
- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;
- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months;
- j. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months;
- k. Not used;
- l. Not used; and
- m. Any other area of station operation considered appropriate by the NSARC or the Senior Vice President.

## ADMINISTRATIVE CONTROLS

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### RECORDS

6.4.3.9 Records of NSARC activities shall be prepared and distributed as indicated below:

- a. Minutes of each NSARC meeting shall be prepared and forwarded to the Senior Vice President within 30 working days following each meeting;
- b. Reports of reviews encompassed by Specification 6.4.2.7 shall be included in the minutes where applicable or forwarded under separate cover to the Senior Vice President within 30 working days following completion of the review; and
- c. Audit reports encompassed by Specification 6.4.2.8 shall be forwarded to the Senior Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

### 6.5 REPORTABLE EVENT ACTION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the results of this review shall be submitted to the NSARC and the Executive Director-Nuclear Production.

### 6.6 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Executive Director-Nuclear Production and the NSARC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe:  
(1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSARC, and the Executive Director-Nuclear Production within 14 days of the violation; and
- d. Operation of the station shall not be resumed until authorized by the Commission.

6.7 PROCEDURES AND PROGRAMS

6.7.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Not used;
- d. Not used;
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation;
- g. Quality Assurance Program for effluent and environmental monitoring;
- h. Fire Protection Program implementation; and
- i. Technical Specification Improvement Program implementation. #

6.7.2 The Station Manager may designate specific procedures and programs or classes of procedures and programs to be reviewed in accordance with the Station Qualified Reviewer Program in lieu of review by the SORC. The review per the Qualified Reviewer Program shall be in accordance with Specification 6.4.2.

6.7.3 Procedures and programs listed in Specification 6.7.1, and changes thereto, shall be approved by the Station Manager or by cognizant Manager or Directors who are designated as the Approval Authority by the Station Manager, as specified in administrative procedures. The Approval Authority for each procedure and program or class of procedure and program shall be specified in administrative procedures.

6.7.4 Each procedure of Specification 6.7.1, and changes thereto, shall be reviewed by the SORC and shall be approved by the Station Manager, or be reviewed and approved in accordance with the Station Qualified Reviewer Program, prior to implementation. Each procedure of Specification 6.7.1 shall be reviewed periodically as set forth in administrative procedures.

6.7.5 Changes to procedures of Specification 6.7.1 may be made prior to SORC review provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license; and
- c. The change is documented, reviewed by the SORC and approved by the Station Manager, or reviewed and approved in accordance with the Station Qualified Reviewer Program, within 14 days of implementation.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-86  
NORTH ATLANTIC ENERGY SERVICE CORPORATION  
SEABROOK STATION, UNIT NO. 1  
DOCKET NO. 50-443

1.0 INTRODUCTION

By application dated October 7, 1994, North Atlantic Energy Service Corporation (North Atlantic/the licensee) proposed an amendment to the Appendix A Technical Specifications for the Seabrook Station, Unit 1 (Seabrook). The proposed changes would remove from the Technical Specifications (TSs) certain audit responsibilities of the Nuclear Safety Audit Review Committee and certain review responsibilities of the Station Operation Review Committee relating to the Emergency Plan and the Security Plan and their respective implementing procedures. The proposed changes also would delete the requirements relating to written procedures applicable to these plans. The proposed changes are consistent with the guidance of Generic Letter 93-07.

2.0 EVALUATION

North Atlantic has proposed changes to TS 6.4.1.6 and 6.4.3.8 which list the review responsibilities of the Station Operation Review Committee (SORC) and audit responsibilities of the Nuclear Safety Audit Review Committee (NSARC) respectively. The proposed changes would remove requirements to review and audit the Emergency Plan and the Security Plan from the TSs since these requirements duplicate the provisions of 10 CFR Part 50 and 10 CFR Part 73. The review and audit requirements in the TSs would be relocated to the respective plans. North Atlantic also proposed to delete from TS 6.7.1 the requirements relating to written procedures for these plans. The NRC issued guidance to all holders of operating licenses for nuclear power reactors on these proposed changes by Generic Letter 93-07.

Part 50.36 of Title 10 of the Code of Federal Regulations established the regulatory requirements related to the content of TSs. The rule requires that the TSs include items in specific categories, including safety limits, limiting conditions for operation, and surveillance requirements. However, the rule does not specify the particular requirements to be included in the plant TSs. The NRC developed criteria, described in the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (58 FR 39132), to determine which of the design conditions and associated surveillances need to be located in the TSs because the requirement is "necessary to obviate the possibility of an abnormal situation or event giving

rise to an immediate threat to the public health and safety." Briefly, those criteria are: (1) detection of abnormal degradation of the reactor coolant pressure boundary, (2) boundary conditions for design basis accidents and transients, (3) primary success paths to prevent or mitigate design basis accidents and transients, and (4) functions determined to be important to risk or operating experience. The Commission's final policy statement acknowledged that its implementation may result in the relocation of existing TS requirements to licensee controlled documents and programs.

North Atlantic has proposed modifying TS 6.4.1.6 by deleting paragraphs i. and j. which reference the Security Plan and Emergency Plan, respectively, and their implementing procedures under the review responsibilities of the SORC. North Atlantic has also proposed modifying TS 6.4.3.8 by deleting paragraphs k. and l. which reference the Emergency Plan and the Security Plan respectively, and their implementing procedures under the audit responsibilities of the NSARC. The removed audit and review requirements would be relocated to the respective Emergency Plan or the Security Plan. Additionally, North Atlantic has proposed to modify TS 6.7.1 by deleting paragraphs c. and d. which reference written procedures for the implementation of the Security Plan and the Emergency Plan. North Atlantic proposed these changes consistent with the guidance provided in GL 93-07.

The NRC staff has determined that the requirements discussed above relating to the Emergency Plan and the Security Plan do not satisfy any of the previously stated final policy statement criteria which would necessitate that they be included in the TSs. On this basis, the staff concludes that these requirements do not need to be controlled by the TSs. In addition, as stated in GL 93-07, 10 CFR Part 50 and 10 CFR Part 73 include provisions that are sufficient to address these requirements. The staff has concluded that the proposed changes to TS 6.4.1.6, 6.4.3.8, and 6.7.1 are, therefore, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Hampshire and Massachusetts State officials were notified of the proposed issuance of the amendment. The State officials had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes an administrative procedure or requirement. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. De Agazio

Date: May 19, 1995