



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 17, 1999

Mr. R. P. Necci - Vice President
Nuclear Oversight and Regulatory Affairs
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - CORRECTION TO
AMENDMENT NO. 170 (TAC NO. MA4574)

Dear Mr. Necci:

By letter dated May 10, 1999, the NRC issued Amendment No. 170 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated January 18, 1999.

The amendment revised the Millstone Unit 3 licensing basis by modifying Technical Specification (TS) 3/4.2.2 to be in accordance with NRC-approved Westinghouse methodologies for the heat flux hot channel factor - $F_Q(Z)$. In addition, the amendment makes changes to the core operating limits and the analytical methods used to determine core operating limits contained in Section 6.9.1.6.a and b, respectively, by adding, modifying, or deleting references.

It has recently come to the NRC staff's attention that the retyped version of the TS that you provided to the staff contained administrative errors and omissions. Specifically, on page 3/4-7 of the retyped page, action statement c.(2) for TS 3.2.2.1 read:

Reduce THERMAL POWER at least 1% for each 1% $F_Q(Z)$ exceeds the limit within 15 minutes and similarly reduce the Power Range Neutron Flux-High Trip Setpoints within the next 4 hours; subsequent POWER OPERATION may proceed provided the Overpower ΔT Trip Setpoints have been reduced at least ...

This action statement should have read:

Reduce THERMAL POWER at least 1% for each 1% $F_Q(Z)$ exceeds the limit within 15 minutes and similarly reduce the Power Range Neutron Flux-High Trip Setpoints within the next 4 hours; **POWER OPERATION may proceed for up to a total of 72 hours;** subsequent POWER OPERATION may proceed provided the Overpower ΔT Trip Setpoints have been reduced at least ...

The bold text should have been included in the retyped page. The staff has determined that this omission was administrative in nature and does not affect the conclusions made by the staff documented in the May 10, 1999, safety evaluation.

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A second editorial error was identified to the staff on the retyped page 6-20a. Specifically, reference 11, should be a letter from V. L. Rooney (USNRC) dated July 18, 1995, instead of a letter from V. L. Rooney dated July 8, 1995.

The enclosed replacement pages are to be used to replace the incorrect pages provided in the May 10, 1999, letter transmitting Amendment No. 170 to Operating License NPF-49 for the Millstone Nuclear Power Station, Unit No. 3. If you have questions or comments regarding the errors and omission in the retyped TS pages, please contact me at 301-415-1278.

Sincerely,

ORIGINAL SIGNED BY:

John A. Nakoski, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosure: Pages 3/4 2-7 and
6-20a

cc w/encl: See next page

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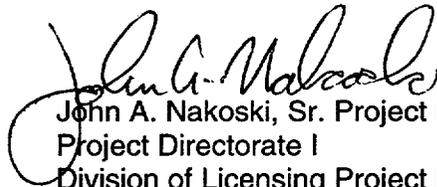
R. Necci

-2-

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John A. Nakoski, Sr. Project Manager, Section 2
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Office of Nuclear Reactor Regulation

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Enclosure: Pages 3/4 2-7 and
6-20a

cc w/encl: See next page

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LIMITING CONDITION FOR OPERATION (Continued)

- (2) Reduce THERMAL POWER at least 1% for each 1% $F_Q(Z)$ exceeds the limit within 15 minutes and similarly reduce the Power Range Neutron Flux-High Trip Setpoints within the next 4 hours; POWER OPERATION may proceed for up to a total of 72 hours; subsequent POWER OPERATION may proceed provided the Overpower ΔT Trip Setpoints have been reduced at least 1% for each 1% $F_Q(Z)$ exceeds its limit shall be calculated as the maximum percent over the core height (Z), consistent with Specification 4.2.2.1.4.f, by the following expression:

$$\left[\left[\frac{F_Q^M(Z) \times W(Z)_{BL}}{F_Q^{RTP}} - 1 \right] \times 100 \text{ for } P \geq APL^{ND} \right]$$

SURVEILLANCE REQUIREMENTS

- 4.2.2.1.1 The provisions of Specification 4.0.4 are not applicable.
- 4.2.2.1.2 For RAOC operation, $F_Q(Z)$ shall be evaluated to determine if $F_Q(Z)$ is within its limit by:
- a. Using the movable incore detectors to obtain a power distribution map at any THERMAL POWER greater than 5% of RATED THERMAL POWER.
 - b. Evaluate the computed heat flux hot channel factor by performing both of the following:
 - (1) Determine the computed heat flux hot channel Factor, $F_Q^M(Z)$ by increasing the measured $F_Q(Z)$ component of the power distribution map by 3% to account for manufacturing tolerances and further increase the value by 5% to account for measurement uncertainties, and
 - (2) Verify that $F_Q^M(Z)$ satisfies the requirements of Specification 3.2.2.1 for all core plane regions, i.e. 0-100% inclusive.

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Cont.)

7. WCAP-11946, "Safety Evaluation Supporting a More Negative EOL Moderator Temperature Coefficient Technical Specification for the Millstone Nuclear Power Station Unit 3," September 1988 (W Proprietary).
8. WCAP-10054-P-A, "WESTINGHOUSE SMALL BREAK ECCS EVALUATION MODEL.17 USING THE NOTRUMP CODE," August 1985 (W Proprietary). (Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
9. WCAP-10079-P-A, "NOTRUMP - A NODAL TRANSIENT SMALL BREAK AND GENERAL NETWORK CODE," August 1985 (W Proprietary). (Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
10. WCAP-12610, "VANTAGE+ Fuel Assembly Report," June 1990 (W Proprietary). (Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
11. Letter from V. L. Rooney (USNRC) to J. F. Opeka, "Safety Evaluation for Topical Report, NUSCO-152, Addendum 4, 'Physics Methodology for PWR Reload Design,' TAC No. M91815," July 18, 1995.
12. Letter from E. J. Mroczka to the USNRC, "Proposed Changes to Technical Specifications, Cycle 4 Reload Submittal - Boron Dilution Analysis," B13678, December 4, 1990.
13. Letter from D. H. Jaffe (USNRC) to E. J. Mroczka, "Issuance of Amendment (TAC No. 77924)," March 11, 1991.
14. Letter from M. H. Brothers to the USNRC, "Proposed Revision to Technical Specification, Shutdown Margin Requirements and Shutdown Margin Monitor Operability for Modes 3, 4, and 5 (PTSCR 3-16-97), B16447, May 9, 1997.
15. Letter from J. W. Anderson (USNRC) to M. L. Bowling (NNECO), "Issuance of Amendment - Millstone Nuclear Power Station, Unit No. 3 (TAC No. M98699)," October 21, 1998.