

July 30, 1993

Docket No. 50-423

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

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Dear Mr. Opeka:

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT 3 - CORRECTION TO AMENDMENT
NO. 80 (TAC NOS. M77362 AND M77432)

On July 12, 1993, the Commission issued Amendment No. 80 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit 3, in response to your application dated March 19, 1993.

The Amendment revised page 3/4 4-39 of the Technical Specifications. When issued, Amendment No. 79 revisions were not incorporated. Enclosed is corrected page 3/4 4-39 incorporating changes issued with Amendment Nos. 79 and 80.

The Safety Evaluation supporting Amendment No. 80 referenced "plant operation in Modes 4, 5 or 6." Enclosed is corrected page 2 of the Safety Evaluation deleting reference to Mode 4.

Sincerely,

Original signed by

Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
Technical Specification
Page 4-39
Safety Evaluation (page 2)

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink, appearing to read "V. Rooney", written over a horizontal line.

Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Technical Specification
Page 4-39
Safety Evaluation (page 2)

cc w/enclosures:
See next page

Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 3

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REACTOR COOLANT SYSTEM

OVERPRESSURE PROTECTION SYSTEM

LIMITING CONDITION FOR OPERATION

ACTION (Continued)

- e. In the event the PORVs, the RHR suction relief valves, or the RCS vent(s) are used to mitigate an RCS pressure transient, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 30 days. The report shall describe the circumstances initiating the transient, the effect of the PORVs, the RHR suction relief valves, or RCS vent(s) on the transient, and any corrective action necessary to prevent recurrence.
- f. Entry into an OPERATIONAL MODE is permitted while subject to these ACTION requirements.

SURVEILLANCE REQUIREMENTS

4.4.9.3.1 Each PORV shall be demonstrated OPERABLE by:

- a. Performance of an ANALOG CHANNEL OPERATIONAL TEST on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required OPERABLE and at least once per 31 days thereafter when the PORV is required OPERABLE;
- b. Performance of a CHANNEL CALIBRATION on the PORV actuation channel at least once per 18 months;* and
- c. Verifying the PORV isolation valve is open at least once per 72 hours when the PORV is being used for overpressure protection.

4.4.9.3.2 Each RHR suction relief valve shall be demonstrated OPERABLE when the RHR suction relief valves are being used for cold overpressure protection as follows:

- a. For RHR suction relief valve 3RHS*RV8708A, by verifying at least once per 12 hours that 3RHS*MV8701A and 3RHS*MV8701C are open;
- b. For RHR suction relief valve 3RHS*RV8708B, by verifying at least once per 12 hours that 3RHS*MV8702B and 3RHS*MV8702C are open; and
- c. Testing pursuant to Specification 4.0.5.

*Except that the surveillance requirement due no later than June 13, 1993, may be deferred until the next refueling outage, but no later than September 30, 1993, whichever is earlier.

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By letter dated March 19, 1993, the Northeast Nuclear Energy Company proposed changes to the Millstone 3 technical specifications in response to Generic Letter 90-06. The licensee did not adopt the staff position for GI 70. Rather, they proposed an alternative to the staff suggested technical specifications. The staff has not yet reached a conclusion as to the acceptability of the licensee's position on GI 70. The proposed technical specifications addressing GI 70 will be dealt with at a later time. The licensee also proposed changes describing the reactor coolant system vent paths, which the staff has not yet reviewed. This evaluation will address only the changes related to GI 94.

The actions proposed by the NRC staff to improve the availability of the low-temperature overpressure protection (LTOP) system represents a substantial increase in the overall protection of the public health and safety and a determination has been made that the attendant costs are justified in view of this increased protection. The technical findings and the regulatory analysis related to Generic Issue 94 are discussed in NUREG-1326, "Regulatory Analysis for the Resolution of Generic Issue 94, Additional Low-Temperature Overpressure Protection for Light-Water Reactors."

The proposed changes to the Millstone 3 technical specifications in the licensee's letter of March 19, 1993, are consistent with that proposed in the staff's generic letter. The proposed modifications to the technical specifications involves plant operation in Modes 5 or 6 with an inoperable LTOP channel. The licensee has adopted the staff position in that operations under such conditions be limited to 24 hours before restoring the LTOP channel to operable status.

The staff has reviewed the licensee's proposed modifications to the Millstone 3 technical specifications. Since the proposed modifications are consistent with the staff's position previously stated in the generic letter and justified in the above mentioned regulatory analysis, the staff finds the proposed modifications to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards