

October 25, 1993

Docket No. 50-423

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M84264)

The Commission has issued the enclosed Amendment No. 84 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated July 31, 1992.

The amendment removes requirements from the License and Technical Specifications pertaining to the Fire Protection Program, and places these same requirements in operating procedures.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9310290116 931025
PDR ADOCK 05000423
P PDR

Enclosures:

1. Amendment No. 84 to NPF-49
2. Safety Evaluation

cc w/enclosures:
See next page

NRC FILE CENTER COPY

DISTRIBUTION:

Docket File	SNorris	CGrimes	LDoerflein	RGI
NRC & Local PDRs	VRooney	ASingh		
PDI-4 Plant	OGC	ACRS (10)		
SVarga	DHagan	OPA		
JCalvo	GHill (2)	OC/LFDCB		

CP-1

OFFICE	LA:PDI-4	PM:PDI-4	D:PDI-4	OGC	
NAME	SNorris	VRooney:bp	JStoltz	A. J. Jensen	
DATE	9/22/93	9/22/93	9/22/93	9/30/93	1/1

OFFICIAL RECORD COPY

Document Name: G:\ROONEY\M84264.AMD 70019

DFOL
1/1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 25, 1993

Docket No. 50-423

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M84264)

The Commission has issued the enclosed Amendment No. 84 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated July 31, 1992.

The amendment removes requirements from the License and Technical Specifications pertaining to the Fire Protection Program, and places these same requirements in operating procedures.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "V. Rooney".

Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.84 to NPF-49
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 3

cc:

Gerald Garfield, Esquire
Day, Berry and Howard
Counselors at Law
City Place
Hartford, Connecticut 06103-3499

R. M. Kacich, Director
Nuclear Licensing
Northeast Utilities Service Company
Post Office Box 270
Hartford, Connecticut 06141-0270

W. D. Romberg, Vice President
Nuclear Operations Services
Northeast Utilities Service Company
Post Office Box 270
Hartford, Connecticut 06141-0270

J. P. Stetz, Vice President
Haddam Neck Plant
Connecticut Yankee Atomic Power Company
362 Injun Hollow Road
East Hampton, Connecticut 06424-3099

Kevin T. A. McCarthy, Director
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm Street
Post Office Box 5066
Hartford, Connecticut 06102-5066

Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Allan Johanson, Assistant Director
Office of Policy and Management
Policy Development and Planning Division
80 Washington Street
Hartford, Connecticut 06106

First Selectmen
Town of Waterford
Hall of Records
200 Boston Post Road
Waterford, Connecticut 06385

S. E. Scace, Vice President
Millstone Nuclear Power Station
Northeast Nuclear Energy Company
Post Office Box 128
Waterford, Connecticut 06385

P. D. Swetland, Resident Inspector
Millstone Nuclear Power Station
c/o U.S. Nuclear Regulatory Commission
Post Office Box 513
Niantic, Connecticut 06357

F. R. Dacimo, Nuclear Unit Director
Millstone Unit No. 3
Northeast Nuclear Energy Company
Post Office Box 128
Waterford, Connecticut 06385

M. R. Scully, Executive Director
Connecticut Municipal Electric
Energy Cooperative
30 Stott Avenue
Norwich, Connecticut 06360

Burlington Electric Department
c/o Robert E. Fletcher, Esq.
271 South Union Street
Burlington, Vermont 05402

David W. Graham
Fuel Supply Planning Manager
Massachusetts Municipal Wholesale
Electric Company
Post Office Box 426
Ludlow, Massachusetts 01056

Nicholas S. Reynolds
Winston & Strawn
1400 L Street, NW
Washington, DC 20005-3502

J. M. Solymossy, Director
Nuclear Quality and Assessment Services
Northeast Utilities Service Company
Post Office Box 270
Hartford, Connecticut 06141-0270



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated July 31, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 84, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The license is further amended by deleting paragraph 2.C.(3)*, and by adding paragraph 2.H* to read as follows:

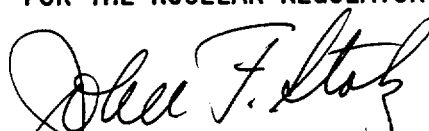
H. Fire Protection (Section 9.5.1, SER, SSER2, SSER4, SSER5)

Northeast Nuclear Energy Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER (NUREG-1031) issued July 1984 and Supplements Nos. 2, 4, and 5 issued September 1985, November 1985 and January 1986, respectively, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John P. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments:

1. Pages 3, 8, and 9 of License
2. Changes to the Technical Specifications

Date of Issuance: October 25, 1993

*Pages 3, 8 and 9 are attached, for convenience, for the composite license to reflect this change.

- (3) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required:
- (4) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) NNECO, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provision of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Northeast Nuclear Energy Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The technical specifications contained in Appendix A revised through Amendment No. , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Northeast Nuclear Energy Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) DELETED

(4) Salem ATWS Events Generic Letter 83-28

NNECO shall submit responses to and implement the requirements of Generic Letter 83-28 Salem ATWS Events on a schedule which is consistent with that given in their November 8, 1983 letter as modified by their letters dated March 16, 1984 and September 5, 1985.

- D. Exemptions from certain requirements of Appendix J 10 CFR Part 50 (Section 6.2.6, SSER 4) and from a portion of the requirements of General Design Criterion 4 (Section 3.9.3.1, SSER 4) of Appendix A to 10 CFR Part 50 have previously been granted. See Safety Evaluation Report Supplement 4, November 1985. With these exemptions the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.
- E. The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Millstone Nuclear Power Station Physical Security Plan," with revisions submitted through March 29, 1988; "Millstone Nuclear Power Station Suitability, Training and Qualification Plan," with revision submitted through July 21, 1986; and "Millstone Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through October 30, 1985. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, Northeast Nuclear Energy Company shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c) and (e).
- G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- H. Fire Protection (Section 9.5.1, SER, SSER 2, SSER4, SSER5)
- Northeast Nuclear Energy Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER (NUREG-1031) issued July 1984 and Supplements Nos. 2, 4, and 5 issued September 1985, November 1985 and January 1986, respectively, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- I. This license is effective as of the date of issuance and shall expire at Midnight on November 25, 2025.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by H. R. Denton

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Attachments/Appendices:

1. Appendix A - Technical Specifications (NUREG-1176)
2. Appendix B - Environmental Protection Plan

Date of Issuance: January 31, 1986

ATTACHMENT TO LICENSE AMENDMENT NO.84

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

i
vi
x
xv
1-3
3/4 3-64
3/4 3-65
3/4 3-66
3/4 3-67
3/4 7-30
3/4 7-31
3/4 7-32
3/4 7-33
3/4 7-33
3/4 7-36
3/4 7-37
3/4 7-38
3/4 7-39
3/4 7-40
3/4 7-41
3/4 7-42
3/4 7-43
B3/4 3-5
B3/4 7-7
B3/4 7-8
6-2
6-5
6-6
6-8

Insert

i
vi
x
xv
1-3
3/4 3-64
3/4 3-65
3/4 3-66
3/4 3-67
3/4 7-30
3/4 7-31
3/4 7-32
3/4 7-33
3/4 7-33
3/4 7-36
3/4 7-37
3/4 7-38
3/4 7-39
3/4 7-40
3/4 7-41
3/4 7-42
3/4 7-43
B3/4 3-5
B3/4 7-7
B3/4 7-8
6-2
6-5
6-6
6-8

INDEX

DEFINITIONS

<u>SECTION</u>	<u>PAGE</u>
<u>1.0 DEFINITIONS</u>	
1.1 ACTION.....	1-1
1.2 ACTUATION LOGIC TEST.....	1-1
1.3 ANALOG CHANNEL OPERATIONAL TEST.....	1-1
1.4 AXIAL FLUX DIFFERENCE.....	1-1
1.5 CHANNEL CALIBRATION.....	1-1
1.6 CHANNEL CHECK.....	1-1
1.7 CONTAINMENT INTEGRITY.....	1-2
1.8 CONTROLLED LEAKAGE.....	1-2
1.9 CORE ALTERATIONS.....	1-2
1.10 DOSE EQUIVALENT I-131.....	1-2
1.11 \bar{E} - AVERAGE DISINTEGRATION ENERGY.....	1-2
1.12 ENCLOSURE BUILDING INTEGRITY.....	1-3
1.13 ENGINEERED SAFETY FEATURES RESPONSE TIME.....	1-3
1.14 DELETED	
1.15 FREQUENCY NOTATION.....	1-3
1.16 IDENTIFIED LEAKAGE.....	1-3
1.17 MASTER RELAY TEST.....	1-3
1.18 MEMBER(S) OF THE PUBLIC.....	1-4
1.19 OPERABLE - OPERABILITY.....	1-4
1.20 OPERATIONAL MODE - MODE.....	1-4
1.21 PHYSICS TESTS.....	1-4
1.22 PRESSURE BOUNDARY LEAKAGE.....	1-4
1.23 PURGE - PURGING.....	1-4
1.24 QUADRANT POWER TILT RATIO.....	1-5
1.25 RADIOACTIVE WASTE TREATMENT SYSTEMS.....	1-5
1.26 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMDCM).....	1-5
1.27 RATED THERMAL POWER.....	1-5
1.28 REACTOR TRIP SYSTEM RESPONSE TIME.....	1-5
1.29 REPORTABLE EVENT.....	1-5
1.30 SHUTDOWN MARGIN.....	1-5
1.31 SITE BOUNDARY.....	1-6

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
TABLE 3.3-5 ENGINEERED SAFETY FEATURES RESPONSE TIMES.....	3/4 3-32
TABLE 4.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-36
3/4.3.3 MONITORING INSTRUMENTATION Radiation Monitoring for Plant Operations.....	3/4 3-42
TABLE 3.3-6 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS.....	3/4 3-43
TABLE 4.3-3 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS SURVEILLANCE REQUIREMENTS.....	3/4 3-45
Movable Incore Detectors.....	3/4 3-46
Seismic Instrumentation.....	3/4 3-47
TABLE 3.3-7 SEISMIC MONITORING INSTRUMENTATION.....	3/4 3-48
TABLE 4.3-4 SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-49
Meteorological Instrumentation.....	3/4 3-50
TABLE 3.3-8 METEOROLOGICAL MONITORING INSTRUMENTATION.....	3/4 3-51
TABLE 4.3-5 METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-52
Remote Shutdown Instrumentation.....	3/4 3-53
TABLE 3.3-9 REMOTE SHUTDOWN INSTRUMENTATION.....	3/4 3-54
TABLE 4.3-6 REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-58
Accident Monitoring Instrumentation.....	3/4 3-59
TABLE 3.3-10 ACCIDENT MONITORING INSTRUMENTATION.....	3/4 3-60
TABLE 4.3-7 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-62
TABLE 3.3-11 DELETED Loose-Part Detection System.....	3/4 3-68
Radioactive Liquid Effluent Monitoring Instrumentation..	3/4 3-69
TABLE 3.3-12 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION	3/4 3-70
TABLE 4.3-8 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-72
Radioactive Gaseous Effluent Monitoring Instrumentation.	3/4 3-74

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
TABLE 3.7-3 STEAM LINE SAFETY VALVES PER LOOP.....	3/4 7-3
Auxiliary Feedwater System.....	3/4 7-4
Demineralized Water Storage Tank.....	3/4 7-6
Specific Activity.....	3/4 7-7
TABLE 4.7-1 SECONDARY COOLANT SYSTEM SPECIFIC ACTIVITY SAMPLE AND ANALYSIS PROGRAM.....	3/4 7-8
Main Steam Line Isolation Valves.....	3/4 7-9
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION.....	3/4 7-10
3/4.7.3 REACTOR PLANT COMPONENT COOLING WATER SYSTEM.....	3/4 7-11
3/4.7.4 SERVICE WATER SYSTEM.....	3/4 7-12
3/4.7.5 ULTIMATE HEAT SINK.....	3/4 7-13
3/4.7.6 FLOOD PROTECTION.....	3/4 7-14
3/4.7.7 CONTROL ROOM EMERGENCY VENTILATION SYSTEM.....	3/4 7-15
3/4.7.8 CONTROL ROOM ENVELOPE PRESSURIZATION SYSTEM.....	3/4 7-18
3/4.7.9 AUXILIARY BUILDING FILTER SYSTEM.....	3/4 7-20
3/4.7.10 SNUBBERS.....	3/4 7-22
FIGURE 4.7-1 SAMPLE PLAN 2) FOR SNUBBER FUNCTIONAL TEST.....	3/4 7-27
3/4.7.11 SEALED SOURCE CONTAMINATION.....	3/4 7-28
3/4.7.12 DELETED	
TABLE 3.7-4 DELETED	
TABLE 3.7-5 DELETED	
3/4.7.13 DELETED	
3/4.7.14 AREA TEMPERATURE MONITORING.....	3/4 7-44
TABLE 3.76 AREA TEMPERATURE MONITORING.....	3/4 7-45

INDEX

BASIS

<u>SECTION</u>	<u>PAGE</u>
3/4.7.11 SEALED SOURCE CONTAMINATION.....	B 3/4 7-6
3/4.7.12 DELETED	
3/4.7.13 DELETED	
3/4.7.14 AREA TEMPERATURE MONITORING.....	B 3/4 7-8
<u>3/4.8 ELECTRICAL POWER SYSTEMS</u>	
3/4.8.1, 3/4.8.2, and 3/4.8.3 A.C. SOURCES, D.C. SOURCES, AND ONSITE POWER DISTRIBUTION.....	B 3/4 8-1
3.4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES.....	B 3/4 8-3
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 BORON CONCENTRATION.....	B 3/4 9-1
3/4.9.2 INSTRUMENTATION.....	B 3/4 9-1
3/4.9.3 DECAY TIME.....	B 3/4 9-1
3/4.9.4 CONTAINMENT BUILDING PENETRATIONS.....	B 3/4 9-1
3/4.9.5 COMMUNICATIONS.....	B 3/4 9-1
3/4.9.6 REFUELING MACHINE.....	B 3/4 9-2
3/4.9.7 CRANE TRAVEL - SPENT FUEL STORAGE AREAS.....	B 3/4 9-2
3/4.9.8 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION.....	B 3/4 9-2
3/4.9.9 CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM.....	B 3/4 9-2
3/4.9.10 and 3/4.9.11 WATER LEVEL - REACTOR VESSEL AND STORAGE POOL.....	B 3/4 9-3
3/4.9.12 FUEL BUILDING EXHAUST FILTER SYSTEM.....	B 3/4 9-3
<u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 SHUTDOWN MARGIN.....	B 3/4 10-1
3/4.10.2 GROUP HEIGHT, INSERTION, AND POWER DISTRIBUTION LIMITS.	B 3/4 10-1
3/4.10.3 PHYSICS TESTS.....	B 3/4 10-1
3/4.10.4 REACTOR COOLANT LOOPS.....	B 3/4 10-1
3/4.10.5 POSITION INDICATION SYSTEM - SHUTDOWN.....	B 3/4 10-1

DEFINITIONS

ENCLOSURE BUILDING INTEGRITY

1.12 ENCLOSURE BUILDING INTEGRITY shall exist when:

- a. Each door in each access opening is closed except when the access opening is being used for normal transit entry and exit,
- b. The Supplementary Leak Collection and Release System is OPERABLE, and
- c. The sealing mechanism associated with each penetration (e.g., welds, bellows, or O-rings) is OPERABLE.

ENGINEERED SAFETY FEATURES RESPONSE TIME

1.13 The ENGINEERED SAFETY FEATURES (ESF) RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF Actuation Setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

1.14 Deleted

FREQUENCY NOTATION

1.15 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.1.

IDENTIFIED LEAKAGE

1.16 IDENTIFIED LEAKAGE shall be:

- a. Leakage (except CONTROLLED LEAKAGE) into closed systems, such as pump seal or valve packing leaks that are captured and conducted to a sump or collecting tank, or
- b. Leakage into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of Leakage Detection Systems or not to be PRESSURE BOUNDARY LEAKAGE, or
- c. Reactor Coolant System leakage through a steam generator to the Secondary Coolant System.

MASTER RELAY TEST

1.17 A MASTER RELAY TEST shall be the energization of each master relay and verification of OPERABILITY of each relay. The MASTER RELAY TEST shall include a continuity check of each associated slave relay.

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

This page intentionally left blank

INSTRUMENTATION

BASES

REMOTE SHUTDOWN INSTRUMENTATION (Continued)

instrumentation, control, and power circuits and transfer switches necessary to eliminate effects of the fire and allow operation of instrumentation, control and power circuits required to achieve and maintain a safe shutdown condition are independent of areas where a fire could damage systems normally used to shut down the reactor. This capability is consistent with General Design Criterion 3 and Appendix R to 10 CFR Part 50.

3/4.3.3.6 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. The instrumentation included in this specification are those instruments provided to monitor key variables, designated as Category 1 instruments following the guidance for classification contained in Regulatory Guide 1.97, Revision 2, "Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident."

In the event more than four sensors in a Reactor Vessel Level channel are inoperable, repairs may only be possible during the next refueling outage. This is because the sensors are accessible only after the missile shield and reactor vessel head are removed. It is not feasible to repair a channel except during a refueling outage when the missile shield and reactor vessel head are removed to refuel the core. If only one channel is inoperable, it should be restored to OPERABLE status in a refueling outage as soon as reasonably possible. If both channels are inoperable, at least one channel shall be restored to OPERABLE status in the nearest refueling outage.

3/4.3.3.7 Deleted.

PLANT SYSTEMS

BASES

3/4.7.11 SEALED SOURCE CONTAMINATION (Continued)

plutonium. This limitation will ensure that leakage from Byproduct, Source, and Special Nuclear Material sources will not exceed allowable intake values.

Sealed sources are classified into three groups according to their use, with Surveillance Requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e., sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

3/4.7.12 Deleted

PLANT SYSTEMS

BASES

3/4.7.13 Deleted

3/4.7.14 AREA TEMPERATURE MONITORING

The area temperature limitations ensure that safety-related equipment will not be subjected to temperatures in excess of their environmental qualification temperatures. Exposure to excessive temperatures may degrade equipment and can cause a loss of its OPERABILITY. The temperature limits include an allowance for instrument error of $\pm 2.2^{\circ}\text{F}$.

ADMINISTRATIVE CONTROLS

FACILITY STAFF (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;
- c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. A Health Physics Technician* shall be on site when fuel is in the reactor;
- e. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or licensed Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- f. Deleted
- g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions. These procedures should follow the general guidance of the NRC Policy Statement on working hours (Generic Letter No. 82-12).

*The Health Physics Technician composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Health Physics Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, May 1977.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Station Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.59. The Director-Nuclear Training has the overall responsibility for the implementation of the Training Program.

6.4.2 Deleted.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Nuclear Unit Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairperson:	Nuclear Unit Director
Vice Chairperson and Member:	Operations Manager
Member:	Maintenance Manager
Member:	Instrument and Controls Manager
Member:	Reactor Engineer
Member:	Radiation Protection Supervisor or Chemistry Supervisor
Member:	Engineering Manager
Member:	Staff Engineer

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PORC Chairperson to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairperson.

QUORUM

6.5.1.5 The quorum of the PORC shall consist of the Chairperson or Vice Chairperson or Nuclear Station Director and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

- a. Review of: (1) all procedures, except common site procedures, required by Specification 6.8 and changes thereto, and (2) any other proposed procedures or changes thereto as determined by the Nuclear Unit Director to affect nuclear safety;
- b. Review of all proposed tests and experiments that affect nuclear safety;
- c. Review of all proposed changes to Sections 1.0-5.0 of these Technical Specifications;
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Executive Vice President-Nuclear and to the Chairperson of the Nuclear Review Board;
- f. Review of all REPORTABLE EVENTS;
- g. Review of facility operations to detect potential safety hazards;
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Chairperson of the Nuclear Review Board or the Nuclear Station Director; and
- i. Render determinations in writing if any item considered under Specification 6.5.1.6a. through d. above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92, constitutes an unreviewed safety question, or requires a significant hazards consideration determination.
- j. Review of Unit Turbine Overspeed Protection Maintenance and Testing Program and revision thereto.
- k. Review of the Fire Protection Program and implementing procedures.

QUORUM

6.5.2.5 A quorum of the SORC shall consist of the Chairperson or Vice Chairperson and five members including alternates.

RESPONSIBILITIES

6.5.2.6 The SORC shall be responsible for:

- a. Review of (1) all common site procedures required by Specification 6.8 and changes thereto, (2) any other proposed procedures or changes thereto as determined by the Nuclear Station Director to affect site nuclear safety;
- b. Review of all proposed changes to Section 6.0 "Administrative Controls" of these Technical Specifications;
- c. Performance of special reviews and investigations and reports as requested by the Chairperson of the Site Nuclear Review Board;
- d. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the Chairperson of the Site Nuclear Review Board;
- e. Review of the Emergency Plan and implementing procedures, and submittal of recommended changes to the Chairperson of the Site Nuclear Review Board;
- f. Review of all common site proposed tests and experiments that affect nuclear safety;
- g. Review of all common site proposed changes or modifications to systems or equipment that affect nuclear safety; and
- h. Render determinations in writing or meeting minutes if any item considered under Specification 6.5.2.6(a) through (g) above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92 constitutes an unreviewed safety question or requires a significant hazards consideration determination.
- i. Review of the common site Fire Protection Program and implementing procedures.

AUTHORITY

6.5.2.7 The SORC shall:

- a. Recommend to the Nuclear Station Director written approval or disapproval in meeting minutes of items considered under Specification 6.5.2.6(a) through (g) above, and
- b. Provide immediate written notification or meeting minutes to the Executive Vice President-Nuclear and the Chairperson of the Site Nuclear Review Board of disagreement between the SORC and the Nuclear Station Director; however, the Nuclear Station Director shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.2.8 The SORC shall maintain written minutes of each meeting and copies shall be provided to the Executive Vice President-Nuclear and Chairperson of the Site Nuclear Review Board.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 84

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated July 31, 1992, the Northeast Nuclear Energy Company (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 Facility Operating License No. NPF-49 and the attached Technical Specifications (TS) and operating licensee. The requested changes would remove requirements from the license and Technical Specifications pertaining to the Fire Protection Program, and place these same requirements in operating procedures.

2.0 EVALUATION

The proposed changes would relocate the Millstone 3 fire protection requirements from Millstone 3 TS to the Millstone 3 fire protection program in accordance with the guidance provided in Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," April 24, 1986, and GL-88-12, "Removal of Fire Protection Requirements from Technical Specifications," August 2, 1988. The following evaluation is based on the staff's review of the letter of July 31, 1992, and the supporting materials provided with the letter.

The TS changes proposed by NNECO are as follows:

1. Delete the definition of fire protection operational test from TS 1.14 and incorporate into Operating Procedure OP-3273, Revision 0, "Technical Requirement Manual" (TRM).
2. Delete TS 3.3.3.7 for fire detection instrumentation and Table 3.3-11 and its associated bases and incorporate into OP-3273.
3. Delete TSs 3.7.12.1, 3.7.12.2, 3.7.12.3, 3.7.12.4, 3.7.12.5, and 3.7.12.6 and their associated bases for fire suppression water system, spray and/or sprinkler systems, carbon dioxide systems, halon systems, fire hose stations, and yard fire hydrants and incorporate into OP-3273.

4. Delete TS 3.7.13 and its associated bases for fire rated assemblies and incorporate into OP-3273.
5. Delete TS 6.2.2.F for the site fire brigade and incorporate into OP-3273.
6. Delete TS 6.4.2 requirements related to the fire brigade training program and incorporate into OP-3273.
7. Add TS 6.5.1.6 (k) to include the review of the fire protection program and implementing procedures as an additional responsibility of the Plant Operations Review Committee (PORC).
8. Add TS 6.5.2.6 (i) to include responsibility for the review of the common site fire protection program and implementing procedures by the Site Operations Review Committee (SORC).

The licensee also proposed the following fire protection license condition:

Northeast Nuclear Energy Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER (NUREG-1031) issued July 1984 and Supplements Nos. 2, 4, and 5 issued September 1985, November 1985, and January 1986, respectively, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

The licensee retained the TS related to the capability for safe shutdown following a fire in the TS.

The NRC staff reviewed the license amendment request for Millstone 3 against the guidance provided in GLs 86-10 and 88-12. Generic Letter 86-10 requested that licensees incorporate the NRC-approved fire protection program in their Final Safety Analysis Report (FSAR). Generic Letter 88-12 addressed the elements a licensee should include in a license amendment request to remove fire protection requirements from TS. These elements are (1) the NRC-approved fire protection program must be incorporated into the FSAR; (2) the Limiting Conditions of Operations and Surveillance Requirements associated with fire detection systems, fire suppression systems, fire barriers, and the administrative controls that address fire brigade staffing would be deleted from the TS (the existing administrative controls related to fire protection audits would be retained in the TS); (3) all operational conditions, remedial actions, and test requirements presently included in the TS for these systems, as well as the fire brigade staffing requirements, shall be incorporated into the fire protection program; (4) the standard fire protection license condition in GL 86-10 must be included in the license; (5) the Unit Review Group (Onsite Review Group) shall be given responsibility for the review of

the fire protection program and implementing procedures and for the submittal of recommended changes to the Company Nuclear Review and Audit group (Off-site or Corporate Review Group); and (6) fire protection program implementation shall be added to the list of elements for which written procedures shall be established, implemented, and maintained.

The licensee incorporated the Millstone 3 fire protection program into its FSAR in January 1986. The current TS operability and surveillance requirements for the detection systems, fire suppression systems, fire rated assemblies, and the fire brigade staffing requirements will be incorporated into the operating procedures and the FSAR. TS 6.5.2.6 (i) will be added to include the responsibility for review of the common site fire protection program and implementing procedures by the SORC. Therefore, the licensee has satisfied elements 1 through 3 of GL 88-12. The licensee proposed the standard fire protection license condition specified in GL 86-10 for Millstone 3. Therefore, the licensee has met element 4 of GL 88-12.

Elements 5 and 6 of GL 88-12 addressed changes to the administrative controls section of the TS. To satisfy these elements, the licensee proposed to add TS 6.5.1.6 (k) to include the review of the fire protection program and implementing procedures as an additional responsibility of the PORC and TS 6.5.2.6 (i) to include the responsibility for the review of the common site fire protection program and implementing procedures by the SORC. Element 6 of GL 88-12 also addressed adding fire protection program implementation to the list of elements for which written procedures shall be established, implemented, and maintained. TS 6.8.1 (g) already addresses the fire protection program, therefore, no changes are required. Therefore, the licensee has satisfied elements 5 and 6 GL 88-12.

The licensee's TS amendment request for Millstone 3 is in accordance with NRC guidance provided in GLs 86-10 and 88-12 and is, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The amendment also relates to changes in recordkeeping or administrative procedures or requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 40217). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Singh

Date: October 25, 1993