1997 September

Mr. Neil S. Carns Senior Vice President and Chief Nuclear Officer Northeast Nuclear Energy Company c/o Ms. Patricia A. Loftus Director - Regulatory Affairs P.O. Box 128 Waterford, CT 06385

SUBJECT:

ISSUANCE OF AMENDMENT (TAC NO. M98724)

Dear Mr. Carns:

The Commission has issued the enclosed Amendment No. 149 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3. in response to your application dated May 14, 1997, as supplemented by letter dated July 30, 1997.

Technical Specification Surveillance Requirement 4.8.2.1.c.4 requires that each battery charger be tested to verify that it can supply a specified current at 125 volts. The amendment increases the required test voltage.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely.

Original signed by:

James W. Andersen, Project Manager Special Projects Office - Licensing Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 149 to NPF-49

2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 5, 1997

Mr. Neil S. Carns
Senior Vice President
and Chief Nuclear Officer
Northeast Nuclear Energy Company
c/o Ms. Patricia A. Loftus
Director - Regulatory Affairs
P.O. Box 128
Waterford, CT 06385

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2. Safety Evaluation

cc w/encls: See next page

Northeast Nuclear Energy Company Station

cc:

Lillian M. Cuoco, Esquire Senior Nuclear Counsel Northeast Utilities Service Company P. O. Box 270 Hartford, CT 06141-0270

Mr. Kevin T. A. McCarthy, Director Monitoring and Radiation Division Department of Environmental Protection 79 Elm Street Hartford, CT 06106-5127

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

First Selectmen Town of Waterford Hall of Records 200 Boston Post Road Waterford, CT 06385

Mr. Wayne D. Lanning Deputy Director of Inspections Special Projects Office 475 Allendale Road King of Prussia, PA 19406-1415

Michael H. Brothers Vice President - Millstone Unit 3 Northeast Nuclear Energy Company P. O. Box 128 Waterford, CT 06385

Mr. M. R. Scully, Executive Director Connecticut Municipal Electric Energy Cooperative 30 Stott Avenue Norwich, CT 06360

Mr. Mr. K. Thayer
Recovery Officer - Nuclear Engineering
and Support
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, Connecticut 06385

Millstone Nuclear Power

Unit 3

Mr. William D. Meinert Nuclear Engineer Massachusetts Municipal Wholesale Electric Company P. O. Box 426 Ludlow, MA 01056

Joseph R. Egan, Esquire Egan & Associates, P.C. 2300 N Street, NW Washington, D.C. 20037

Mr. F. C. Rothen
Vice President - Nuclear Work Services
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Ernest C. Hadley, Esquire 1040 B Main Street P. O. Box 549 West Wareham, MA 02576

Mr. John Buckingham
Department of Public Utility Control
Electric Unit
10 Liberty Square
New Britain, CT 06051

Mr. James S. Robinson
Manager, Nuclear Investments and
Administration
New England Power Company
25 Research Drive
Westborough, MA 01582

Mr. D. M. Goebel Vice President - Nuclear Oversight Northeast Nuclear Energy Company P. O. Box 128 Waterford, CT 06385 Northeast Nuclear Energy Company Station

cc:

Deborah Katz, President Citizens Awareness Network P. O. Box 83 Shelburne Falls, MA 03170

Senior Resident Inspector
Millstone Nuclear Power Station
c/o U.S. Nuclear Regulatory
Commission
P. O. Box 513
Niantic, CT 06357

Mr. Allan Johanson, Assistant Director Office of Policy and Management Policy Development and Planning Division 450 Capitol Avenue - MS# 52ERN P. O. Box 341441 Hartford, CT 06134-1441

Citizens Regulatory Commission ATTN: Ms. Susan Perry Luxton 180 Great Neck Road Waterford, Connecticut 06385

The Honorable Terry Concannon Co-Chair Nuclear Energy Advisory Council Room 4035 Legislative Office Building Capitol Avenue Hartford, Connecticut 06106

Mr. Evan W. Woollacott Co-Chair Nuclear Energy Advisory Council 128 Terry's Plain Road Simsbury, Connecticut 06070

Little Harbor Consultants, Inc. Millstone - ITPOP Project Office P. O. Box 0630 Niantic, Connecticut 06357-0630 Millstone Nuclear Power

Unit 3

Mr. B. D. Kenyon President and Chief Executive Officer Northeast Nuclear Energy Company P. O. Box 128 Waterford, CT 06385

Mr. Daniel L. Curry Project Director Parsons Power Group Inc. 2675 Morgantown Road Reading, Pennsylvania 19607

Mr. Don Schopfer Verification Team Manager Sargent & Lundy 55 E. Monroe Street Chicago, Illinois 60603



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 149 License No. NPF-49

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated May 14, 1997, as supplemented by letter dated July 30, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 149, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Phillip F. McKee

Deputy Director for Licensing

Special Projects Office

Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 5, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 149

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following page of the Appendix A, Technical Specifications, with the attached page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

<u>Remove</u>

<u>Insert</u>

3/4 8-12

3/4 8-12

SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 92 days and within 7 days after a battery discharge with battery terminal voltage below 110 volts, or battery overcharge with battery terminal voltage above 150 volts, by verifying that:
 - 1) The parameters in Table 4.8-2a meet the Category B limits,
 - There is no visible corrosion at either terminals or connectors, or the connection resistance of these items is less than 150×10^{-6} ohm, and
 - 3) The average electrolyte temperature of six connected cells is above 60°F.
- c. At least once per 18 months by verifying that:
 - 1) The cells, cell plates, and battery racks show no visual indication of physical damage or abnormal deterioration,
 - 2) The cell-to-cell and terminal connections are clean, tight, and coated with anticorrosion material.
 - The resistance of each cell-to-cell and terminal connection is less than or equal to 150×10^{-8} ohm, and
 - 4) Each battery charger will supply at least the amperage indicated in Table 4.8-2b at greater than or equal to 132 volts for at least 24 hours.
- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual or simulated emergency loads for the design duty cycle when the battery is subjected to a battery service test;
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. Once per 60-month interval this performance discharge test may be performed in lieu of the battery service test required by Specification 4.8.2.1d.; and
- f. At least once per 18 months, during shutdown, by giving performance discharge tests of battery capacity to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 149

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

In Licensee Event Report 97-01-00, dated February 3, 1997, Northeast Nuclear Energy Company (the licensee) reported that the battery charger surveillance testing was not being performed in compliance with the 125 Vdc requirement of Technical Specification (TS) Surveillance Requirement 4.8.2.1.c.4. The actual surveillance testing was performed at a voltage greater than 125 Vdc. The licensee committed to submit to the NRC staff any required proposed changes to the TS.

By letter dated May 14, 1997, as supplemented July 30, 1997, the Northeast Nuclear Energy Company, et al., submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3, Technical Specifications (TS). TS Surveillance Requirement 4.8.2.1.c.4 requires that each battery charger be tested to verify that it can supply a specified current at 125 volts. The proposed amendment would increase the required test voltage. The July 30, 1997, letter provided clarifying information that did not change the scope of the May 14, 1997, application and the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

TS Surveillance Requirement 4.8.2.1.c.4 currently requires that each battery charger will supply at least the amperage indicated in Table 4.8-2b at 125 volts for at least 24 hours. Table 4.8-2b lists the applicable battery chargers and their required amperage. In its letter dated May 14, 1997, the licensee stated that the proposed change will modify the surveillance requirement by replacing the current requirement to test at exactly 125 volts with a requirement to test at greater than or equal to 132 volts. The licensee stated that this change will provide assurance that the battery charger is capable of supplying the largest combined demands of the various steady state loads, plus the current required to recharge its battery, which has undergone a duty cycle discharge, to its fully charged condition in less than 24 hours.

Regulatory Guide 1.32, "Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants," requires that the capacity of the battery charger supply should be based on the largest combined demands of the various steadystate loads and the charging capacity to restore the battery from the design minimum charge state to the fully charged state, irrespective of the status of the plant during which these demands occur. The NRC staff has reviewed the licensee's proposed change and supporting documentation and has determined that the change from 125 volts to greater than or equal to 132 volts is acceptable. The staff's determination is based on (1) the proposed voltage of \geq 132 Vdc is compatible with the ratings of the supplied equipment, (2) the proposed voltage is within the manufacturer's recommended float voltage of 132 Vdc to 135 Vdc for these types of batteries, (3) the surveillance requirement continues to provide the necessary assurance that the battery chargers will function as required, and (4) the change meets the intent of Regulatory Guide 1.32. The staff notes that the proposed change is consistent with the Standard Technical Specifications for Westinghouse Plants (NUREG-1431) in that the surveillance requirement requires that each battery charger supply greater than or equal to a specific amperage at greater than or equal to a specific voltage for greater than or equal to a specific time. Therefore, based on the above, the staff finds the proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 33130 dated June 18, 1997). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Andersen

Date: September 5, 1997