

November 29, 1995

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
P.O. Box 270
Hartford, CT 06141-0270

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M92562)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated June 8, 1995.

The amendment modifies Surveillance Requirement (SR) 4.5.1.c and deletes Technical Specification (TS) 3/4.8.4.3, "AC Circuits Inside Containment." The changes to SR 4.5.1.c clarify the requirements for securing the safety injection accumulator isolation valve breakers (3SIL*MV8808A, B, C, and D) in the tripped position for the applicable modes. The amendment also deletes TS 3/4.8.4.3 since reasonable assurance is provided to protect the electrical penetrations and penetration conductors against an overcurrent condition and single failure of a circuit breaker.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Vernon L. Rooney, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket No. 50-423

- Enclosures: 1. Amendment No. 121 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

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*11/22/95 no changes made
OGC concurred*

DOCUMENT NAME: G:\GRIGGS\92562.AMD *See previous concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 29, 1995

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Sincerely,

A handwritten signature in black ink, appearing to read "V. Rooney".

Vernon L. Rooney, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 121 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

J. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated June 8, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 29, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

xi
3/4 5-2
3/4 8-23

Insert

xi
3/4 5-2

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EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 31 days when the RCS pressure is above 1000 psig by verifying that the associated circuit breakers are locked in a deenergized position or removed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 121

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated June 8, 1995, the Northeast Nuclear Energy Company (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 Technical Specifications (TS). The changes would modify Surveillance Requirement (SR) 4.5.1.c and delete TS 3/4.8.4.3, "AC Circuits Inside Containment." The changes to SR 4.5.1.c clarify the requirements for securing the safety injection accumulator isolation valve breakers (3SIL*MV8808A, B, C, and D) in the tripped position for the applicable modes. The amendment also deletes TS 3/4.8.4.3 since reasonable assurance is provided to protect the electrical penetration and penetrations conductors against an overcurrent condition and single failure of a circuit breaker.

2.0 BACKGROUND

TS 3.5.1 requires that electrical power to the safety injection accumulator isolation valves 3SIL*MV8808A, B, C, and D be removed in order for the accumulators to be operable. Surveillance Requirement 4.5.1.c currently requires demonstrating accumulator operability as follows: "At least once per 31 days when the reactor coolant system (RCS) pressure is above 1000 psig by verifying that power to the isolation valve is disconnected by removal of the breaker from the circuit." The proposed TS changes would revise SR 4.5.1.c as follows: "At least once per 31 days when the RCS pressure is above 1000 psig by verifying that the associated circuit breakers are locked in a deenergized position or removed." TS 3.8.4.3 requires removal of electrical power to the safety injection accumulator isolation valves 3SIL*MV8808A, B, C, and D in Modes 1, 2, 3, and 4. SR 4.8.4.3 requires that at least once every 24 hours (or at least once every 31 days if locked, sealed, or otherwise secured in the tripped condition) each of the alternating current circuits for the above-listed valves are in a deenergized state by verifying that the associated circuit breakers are in the tripped position. The proposed TS changes would remove TS 3/4.8.4.3.

3.0 EVALUATION

3.1 Proposed Revisions to SR 4.5.1.c

Power is removed from accumulator isolation valves to prevent the inadvertent closure of these valves, which would block the safety function of the accumulators. The proposed changes to SR 4.5.1.c will clarify the guidance on removing electrical power to the accumulator isolation valves, which is to be accomplished by securing associated breakers in the tripped (off) position when the RCS pressure is above 1000 psig. The revised SR 4.5.1.c will provide clearer guidance to ensure that power to the accumulator isolation valve is removed when the accumulators are required to be operable. The proposed TS change clarifies SR 4.5.1.c and is acceptable.

3.2 Proposed Removal of Technical Specification Section 3/4.8.4.3

The purpose of the requirement to remove electrical power from the accumulator isolation valves in TS 3.8.4.3 is to protect containment electrical penetrations and penetration conductors. The Bases Section 3/4.8.4 states that containment electrical penetration and penetration conductors are protected by either deenergized circuits not required during reactor operation or by demonstrating the operability of primary and backup overcurrent protection circuit breakers during periodic surveillance.

Section 8.3.3.1.1 of the Millstone Unit 3 Safety Evaluation Report (SER), dated July 1984, addressed an issue of submerged electrical equipment as a result of a loss-of-coolant accident. The accumulator isolation motor-operated valves (MOV) 3SIL*MV8808A, B, C, and D were identified as electrical equipment not qualified for submergence and therefore they could introduce faults into the Class 1E buses when submerged. At the time the SER was written, MOVs 3SIL*MV8808A, B, C, and D were not protected by a second breaker. Therefore, the licensee was required by TS to periodically verify that these valves were deenergized during normal plant operation. A design change has been made to add back up electrical penetration breakers to a number of circuits, including MOVs 3SIL*MV8808A, B, C, and D. Both the primary and back up breakers for these MOVs are designed to trip before the penetration thermal capability is exceeded.

Removal of TS Section 3/4.8.4.3 is justified because TS 3/4.5.1, "Accumulators," provides assurance that electrical power to the accumulator isolation valves 3SIL*MV8808A, B, C, and D is removed in Modes 1 through 3. With the addition of the back up breaker, two breakers in series provide redundant protection for the electrical penetrations and penetration conductors against an overcurrent condition. Periodic surveillance testing demonstrates that these two breakers are operable (existing SR 4.8.4.1). On the basis of this information, the staff concludes that the removal of TS 3/4.8.4.3 is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, a good faith effort was made to contact the Connecticut State official about the proposed issuance of the amendment.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 39444). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Nguyen

Date: November 29, 1995