March 2, 1990 \checkmark

Docket No. 50-423

Mr. Edward J. Mroczka Senior Vice President Nuclear Engineering and Operations Connecticut Yankee Atomic Power Company Northeast Nuclear Energy Company Post Office Box 270 Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: MILLSTONE UNIT 3 - ISSUANCE OF AMENDMENT (TAC NOS. 75312 & 75393)

The Commission has issued the enclosed Amendment No. 47 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your applications dated November 2 and December 1, 1989.

The amendment modifies Technical Specification (TS) 3.6.4.1, "Hydrogen Monitors," and TS 3.3.3.6, "Accident Monitoring Instrumentation," to eliminate inconsistencies concerning Limiting Conditions for Operations (LCOs) associated with hydrogen monitors. The amendment also modifies TS 4.6.4.2b.4, "Electric Hydrogen Recombiners," to provide variable acceptance criteria for flow testing.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

David H. Jaffe, Project Manager Project Directorate I-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 47 to NPF-49

2. Safety Evaluation

cc w/enclosures: See next page

OFC :LA:PDI-4: PMAPDI-4	: PD:PDI-4	:OGC ¥	•		:
NAME :SNorris: DJaffe:1m	: JStolz	: BmB	• • • • • • • •	::	•••••••••••••••••••••••••••••••••••••••
DATE :2/12/90: 1/13/90	:2/3/90	:2 //6/90	:	·	·····
OFFICIAL RECORD COPY Document Name: AMEND TAC	75312	H WAUTRERCE Conditioned on Amondment NUT TU ISSUE NUT TU ISSUE			î.7.
9003160383 900302 PDR ADOCK 05000423 PDC PDC		5 willy North Period HAMA Expliced		VFUL 'II	MURP-1



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

March 2, 1990

Docket No. 50-423

Mr. Edward J. Mroczka Senior Vice President Nuclear Engineering and Operations Connecticut Yankee Atomic Power Company Northeast Nuclear Energy Company Post Office Box 270 Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: MILLSTONE UNIT 3 - ISSUANCE OF AMENDMENT (TAC NOS. 75312 & 75393)

The Commission has issued the enclosed Amendment No. 47 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your applications dated November 2 and December 1, 1989.

The amendment modifies Technical Specification (TS) 3.6.4.1, "Hydrogen Monitors," and TS 3.3.3.6, "Accident Monitoring Instrumentation," to eliminate inconsistencies concerning Limiting Conditions for Operations (LCOs) associated with hydrogen monitors. The amendment also modifies TS 4.6.4.2b.4, "Electric Hydrogen Recombiners," to provide variable acceptance criteria for flow testing.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerel

David H. Jaffe, Project Manager Project Directorate I-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures: 1. Amendment No.47 to NPF-49 2. Safety Evaluation

cc w/enclosures: See next page Mr. E. J. Mroczka Northeast Nuclear Energy Company

cc:

Gerald Garfield, Esquire Day, Berry and Howard Counselors at Law City Place Hartford, Connecticut 06103-3499

W. D. Romberg, Vice President Nuclear Operations Northeast Utilities Service Company Post Office Box 270 Hartford, Connecticut 06141-0270

Kevin McCarthy, Director Radiation Control Unit Department of Environmental Protection State Office Building Hartford, Connecticut 06106

Bradford S. Chase, Under Secretary Energy Division Office of Policy and Management 80 Washington Street Hartford, Connecticut 06106

S. E. Scace, Nuclear Station Director Millstone Nuclear Power Station Northeast Nuclear Energy Company Post Office Box 128 Waterford, Connecticut 06385

C. H. Clement, Nuclear Unit Director Millstone Unit No. 3 Northeast Nuclear Energy Company Post Office Box 128 Waterford, Connecticut 06385

Ms. Jane Spector Federal Energy Regulatory Commission 825 N. Capitol Street, N.E. Room 8608C Washington, D.C. 20426

Burlington Electric Department c/o Robert E. Fletcher, Esq. 271 South Union Street Burlington, Vermont 05402 Millstone Nuclear Power Station Unit No. 3

R. M. Kacich, Manager Generation Facilities Licensing Northeast Utilities Service Company Post Office Box 270 Hartford, Connecticut 06141-0270

D. O. Nordquist Director of Quality Services Northeast Utilities Service Company Post Office Box 270 Hartford, Connecticut 06141-0270

Regional Administrator Region I U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406

First Selectmen Town of Waterford Hall of Records 200 Boston Post Road Waterford, Connecticut 06385

W. J. Raymond, Resident Inspector Millstone Nuclear Power Station c/o U. S. Nuclear Regulatory Commission Post Office Box 811 Niantic, Connecticut 06357

M. R. Scully, Executive Director Connecticut Municipal Electric Energy Cooperative
30 Stott Avenue
Norwich, Connecticut 06360

Mr. Alan Menard, Manager Technical Services Massachusetts Municipal Wholesale Electric Company Post Office Box 426 Ludlow, Massachusetts 01056



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47 License No. NPF-49

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated November 2 and December 1, 1989 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.47 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director Project Directorate I-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 2, 1990

- 2

ATTACHMENT TO LICENSE AMENDMENT NO. 47

FACILTIY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove	Insert
ix	ix
3/4 3-59	3/4 3-59
3/4 6-35	3/4 6-35
3/4 6-36	3/4 6-36
-	3/4 6-36a

LIMITING COND	ITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS	-
SECTION		PAGE
FIGURE 3.6-1	MAXIMUM ALLOWABLE PRIMARY CONTAINMENT AIR PARTIAL PRESSURE VERSUS SERVICE WATER TEMPERATURE	3/4 6-8
	Air Temperature	3/4 6-9
	Containment Structural Integrity	3/4 6-10
	Containment Ventilation System	3/4 6-11
3/4.6.2	DEPRESSURIZATION AND COOLING SYSTEMS	
	Containment Quench Spray System	3/4 6-12
	Recirculation Spray System	3/4 6-13
	Spray Additive System	3/4 6-14
3/4.6.3	CONTAINMENT ISOLATION VALVES	3/4 6-15
3/4.6.4	COMBUSTIBLE GAS CONTROL	
	Hydrogen Monitors	3/4 6-35
	Electric Hydrogen Recombiners	3/4 6-36
FIGURE 3.6-2	HYDROGEN RECOMBINER ACCEPTANCE CRITERIA FLOW VS. CONTAINMENT PRESSURE	3/46-36a
3/4.6.5	SUBATMOSPHERIC PRESSURE CONTROL SYSTEM	
	Steam Jet Air Ejector	3/4 6-37
3/4.6.6	SECONDARY CONTAINMENT	
	Supplementary Leak Collection and Release System	3/4 6-38
	Enclosure Building Integrity	3/4 6-40
	Enclosure Building Structural Integrity	3/4 6-41
<u>3/4.7 PLANT SY</u>	STEMS	
3/4.7.1 TURB	INE CYCLE	
Safe	ty Valves	3/4 7-1
TABLE 3.7-1	MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT WITH INOPERABLE STEAM LINE SAFETY VALVES DURING FOUR LOOP OPERATION	3/4 7-2
TABLE 3.7-2	MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT WITH INOPERABLE STEAM LINE SAFETY VALVES THREE LOOP OPERATION	3/4 7-2

MILLSTONE - UNIT 3

· ·

ş

Amendment No. 28, 47

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels except the containment area high range radiation monitor, and the containment hydrogen monitor less than the Total Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels except the containment area-high range radiation monitor, and the containment hydrogen monitor less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- c. With the number of OPERABLE channels for the containment area-high range radiation monitor less than required by either the total or the Minimum Channels OPERABLE requirements, initiate an alternate method of monitoring the appropriate parameter(s), within 72 hours, and either restore the inoperable channel(s) to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission, pursuant to Specification 6.9.2, within 14 days that provides actions taken, cause of the inoperability, and the plans and schedule for restoring the channels to OPERABLE status.
- d. With the number of OPERABLE channels for the containment hydrogen monitors less than the total number of channels shown in Table 3.3-10, restore the inoperable channel to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours. With the number of operable channels for the containment hydrogen monitors less than the minimum channels OPERABLE requirement of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- e. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION at the frequencies shown in Table 4.3-7.

MILLSTONE - UNIT 3

Amendment No. 47

I

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION .

- UNII	INS	TRUMENT	TOTAL NO. OF <u>CHANNELS</u>	MINIMUM CHANNELS OPERABLE
رب س	1.	Containment Pressure	- , ,	· · · ·
		a. Normal Range b. Extended Range	2 2	1
3/4 3-60 Amendment No. 46	2.	Reactor Coolant Outlet Temperature - T _{HOT} (Wide Range)	2	1
	3.	Reactor Coolant Inlet Temperature - T _{COLD} (Wide Range)	2	1
	4.	Reactor Coolant Pressure - Wide Range	2	1
	5.	Pressurizer Water Level	2	-
	6.	Steam Line Pressure	2/steam generator	- 1/steam generator
	7.	Steam Generator Water Level - Narrow Range	l/steam generator	1/steam generator
	8.	Steam Generator Water Level - Wide Range	l/steam generator	1/steam generator
	9.	Refueling Water Storage Tank Water Level	2	1
	10.	Demineralized Water Storage Tank Water Level	2	t
	11.	Auxiliary Feedwater Flow Rate	2/steam generator	1/steam generator
	12.	Reactor Coolant System Subcooling Margin Monitor	2	t
	13.	Containment Water Level (Wide Range)	2	• • •
	14.	Core Exit Thermocouples	4/core quadrant	- 2/core quadmant
	15.	DELETED	, Jaan mit	-y -vie quaurant

MILLSTONE

۰.

CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN MONITORS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two independent containment hydrogen monitors shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTION:

- a. With one hydrogen monitor inoperable, restore the inoperable monitor to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- b. With both hydrogen monitors inoperable, restore at least one monitor to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen monitor shall be demonstrated OPERABLE by the performance of a CHANNEL CHECK at least once per 12 hours, an ANALOG CHANNEL OPERATIONAL TEST at least once per 31 days, and at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gas containing:

- a. One volume percent hydrogen, balance nitrogen, and
- b. Four volume percent hydrogen, balance nitrogen.

CONTAINMENT SYSTEMS

ELECTRIC HYDROGEN RECOMBINERS

LIMITING CONDITION FOR OPERATION

3.6.4.2 Two independent Hydrogen Recombiner Systems shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

With one Hydrogen Recombiner System inoperable, restore the inoperable system to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

- 4.6.4.2 Each Hydrogen Recombiner System shall be demonstrated OPERABLE:
 - a. At least once per 6 months by verifying during a Hydrogen Recombiner System functional test that the minimum reaction chamber gas temperature increases to greater than or equal to 700°F within 90 minutes and is maintained for at least 2 hours and that the purge blower operates for 15 minutes.
 - b. At least once per 18 months by:
 - 1) Performing a CHANNEL CALIBRATION of all recombiner instrumentation and control circuits,
 - Verifying through a visual examination that there is no evidence of abnormal conditions within the recombiner enclosure (i.e., loose wiring or structural connections, deposits of foreign materials, etc.),
 - 3) Verifying the integrity of all heater electrical circuits by performing a resistance to ground test following the above required functional test. The resistance to ground for any heater phase shall be greater than 10,000 ohms, and
 - 4) Verifying during a recombiner system functional test using containment atmospheric air at a flow rate of greater than or equal to that indicated on Figure 3.6-2, that the gas temperature increases to greater than or equal to 1100°F within 5 hours and is maintained for at least 4 hours.

MILLSTONE - UNIT 3

3/4 6-36





3/4 6-36a

.....

Amendment No. 47

MILLSTONE - UNIT 3



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 47

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

INTRODUCTION

9003160390 900302 PDR ADOCK 05000423

PTI

By applications for license amendment dated November 2 and December 1, 1989, Northeast Nuclear Energy Company, et al. (the licensee), requested changes to Millstone Unit 3 Technical Specifications (TS).

The proposed amendment would modify TS 3.6.4.1, "Hydrogen Monitors," and TS 3.3.3.6, "Accident Monitoring Instrumentation," to eliminate inconsistencies concerning Limiting Conditions for Operations (LCOs) associated with hydrogen monitors. The proposed amendment would also modify 4.6.4.2b.4, "Electric Hydrogen Recombiners," to provide variable acceptable criteria for flow testing.

DISCUSSION AND EVALUATION

The operability of the combustible gas control equipment required for the detection and control of hydrogen gas ensures that this equipment will be available to maintain the hydrogen concentration within containment below its flammable limit during post-loss-of-coolant-accident (LOCA) conditions. The hydrogen monitors provide information used to determine the need to start the hydrogen recombiners or initiate containment purge. At the present time, TS 3.6.4 and TS 3.3.3.6 contain conflicting requirements concerning remedial actions to be taken when one or more hydrogen monitors become inoperable, as follows:

- ^o TS 3.3.3.6 allows reactor operation up to 7 days with one hydrogen monitor inoperable and up to 48 hours with two hydrogen monitors inoperable. If the inoperable monitors cannot be returned to service, the plant must be in Hot Standby in 6 hours and Hot Shutdown within the following 6 hours. Operability of the hydrogen monitors is required in Modes 1, 2 and 3. Startup of the facility with inoperable hydrogen monitors is permitted (the provisions of TS 3.0.4 are not applicable.)
- ^o TS 3.6.4.1 allows reactor operation up to 30 days with one hydrogen monitor inoperable and up to 72 hours with two hydrogen monitors inoperable. If the inoperable monitors cannot be returned to service, the plant must be in Hot Standby within 6 hours. Operability of the hydrogen monitors is required in Modes 1 and 2. Startup of the facility with inoperable hydrogen monitors is not permitted (the provisions of TS 3.0.4 are applicable.)

The licensee has proposed that TS 3.6.4.1 and TS 3.3.3.6 be modified to eliminate the conflicting requirements concerning inoperable hydrogen monitors. The requirements of TS 3.3.3.6 would be changed to increase the allowable out-of-service time from 7 days to 30 days for one hydrogen monitor and from 48 hours to 72 hours for inoperability of two hydrogen monitors. The above changes to TS 3.3.3.6 would provide consistency with the requirements of TS 3.6.4.1. The proposed change to TS 3.3.3.6 also includes a reformatting of the associated Action Statements to segregate the LCOs for the hydrogen monitor. The requirements of TS 3.6.4.1 would be changed as follows to provide consistency with TS 3.3.3.6:

- Increase the range of applicable modes from Modes 1 and 2 to Modes 1, 2 and 3.
- [°] A statement would be added that the provisions of TS 3.0.4 are not applicable.
- [°] The remedial action required for the inoperability of one or more hydrogen monitors, in the event that the monitors cannot be restored would be extended from "...at least Hot Standby within the next 6 hours" to also include "...and in at least Hot Shutdown within the following 6 hours."

On November 1, 1983, the NRC staff issued Generic Letter (GL) 83-37, "Technical Specifications." The purpose of GL 83-77 was to provide guidance to licensees on modifications of TS to address a number of Post-TMI concerns including the operability of hydrogen monitors. In this regard, the licensee's proposed changes to TS 3.3.3.6 and 3.6.4.1 are in substantial conformance with NRC staff guidance on operability of hydrogen monitors and are thus, acceptable.

With regard to the hydrogen recombiner system, this system is designed to maintain the hydrogen concentration in the containment below 4 volume percent following a LOCA. To ensure operability of the system in the event of a single failure of any component, the system is arranged in two redundant 100 percent capacity trains. The parameters presented in Regulatory Guide 1.7 are used in the analysis of hydrogen generation following a DBA. The analysis of hydrogen generation following a LOCA and the capability of the hydrogen recombiners or the backup purge system, to maintain a hydrogen concentration below 4 percent volume is described in the Millstone Unit No. 3 Final Safety Analysis Report (FSAR) Section 6.2.5.

Technical Specification 4.6.4.2b.4 requires that the licensee verify, every 18 months, that the hydrogen recombiners demonstrate a flow rate of at least 40 scfm. Although hydrogen recombiner flow rate is a function of containment pressure, the existing TS 4.6.4.2b.4 does not specify a containment pressure for performance of the hydrogen recombiner flow test.

During a recent review of hydrogen recombiner surveillance test data and the original calculation for post LOCA hydrogen generation inside containment, the licensee identified an inconsistency. Specifically, the original analysis assumed that 50 scfm of containment air would flow thorough the system while the containment pressure was at 9 psia in post-LOCA conditions. This assumed

- 2 -

- 3 -

flow rate is also greater than tested flow rates which have been as low as 42 scfm when containment is between 9 and 10 psia. Therefore, the licensee performed a new calculation and determined the minimum acceptable flow rate through hydrogen recombiners to be 40.5 scfm. This flow rate will still ensure containment hydrogen concentration remains below 4 volume percent during the accident if the hydrogen recombiners are started within 24 hours of a DBA when the hydrogen concentration of the containment atmosphere is at or below 1.8 volume percent. As stated in FSAR Section 6.2.5, the hydrogen recombiners would be started well before 1.8 volume percent hydrogen is detected in the containment.

The proposed change to TS 4.6.4.2b.4 would replace the 50 scfm acceptance criteria for the hydrogen recombiner flow test with an acceptance criteria that is a function of containment pressure. The proposed variable acceptance criteria range almost linearly from approximately 40 scfm at a containment pressure of 8.5 psia to approximately 80 scfm at a containment pressure of 15.5 psia. The lowest flow acceptance criterion, however, would be limited to approximately 51 scfm since the lowest containment pressure is limited to 10.6 psia by TS 3.6.4.1 as indicated in the licensee's letter dated January 19, 1990.

The proposed change to TS 4.5.4.2b.4 provides acceptance criteria for hydrogen recombiner testing which are adequate to demonstrate that the recombiners will perform their post-LOCA design function. Moreover, the acceptance criteria provide the flexibility to allow the licensee to test the recombiners over a range of containment pressures. Accordingly, the proposed change to TS 4.6.4.2b.4 is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 2, 1990

Principal Contributor: D. Jaffe

DATED: March 2, 1990 AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-49

DISTRIBUTION Docket File NRC & Local PDR Plant File S. Varga (14E4) B. Boger (14A2) J. Stolz S. Norris D. Jaffe OGC D. Hagan (MNBB 3302) E. Jordan (MNBB 3302) G. Hill(4) (P1-137) W. Jones (P-130A) J. Calvo (11F23) ACRS (10) GPA/PA ARM/LFMB

.