

January 27, 1995

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: CORRECTION TO AMENDMENT NO. 94 (TAC NO. M89530)

On January 3, 1995, the Commission issued Amendment No. 100 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit 3, in response to your application dated June 2, 1994, as supplemented August 25, 1994.

The amendment revised the Technical Specifications (TS) to remove expired one-time extensions of surveillance, removed an obsolete definition of charging pump operability, and incorporated 11 line item improvements in accordance with the guidance provided in Generic Letter 93-05. Several editorial changes were made to renumber TS pages and delete the blank pages from the TS.

It has been brought to our attention that TS pages 3/4 3-12 and 3/4 7-5 issued with Amendment 100 did not incorporate changes made by Amendments 79 and 96, respectively. Enclosed are corrected TS pages 3/4 3-12 and 3/4 7-5. We are sorry for any inconvenience this may have created.

Sincerely,

Original signed by:

Vernon L. Rooney, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosure: Technical Specification Pages 3/4 3-12
and 3/4 7-5

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
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Sincerely,

A handwritten signature in black ink, appearing to read "V. L. Rooney", is written over a horizontal line.

Vernon L. Rooney, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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and 3/4 7-5

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Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 3

cc:

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TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
18. Reactor Trip Breaker	N.A.	N.A.	N.A.	M(7, 11)	N.A.	1, 2, 3*, 4*, 5*
19. Automatic Trip and Interlock Logic	N.A.	N.A.	N.A.	N.A.	M(7)	1, 2, 3*, 4*, 5*
20. Three Loop Operation Bypass Circuitry	N.A.	N.A.	N.A.	R	N.A.	1, 2
21. Reactor Trip Bypass Breaker	N.A.	N.A.	N.A.	M(7, 15) R(16)	N.A.	1, 2, 3*, 4*, 5*
22. Shutdown Margin Monitor	N.A.	N.A.	Q(19)	N.A.	N.A.	3, 4, 5

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 92 days on a STAGGERED TEST BASIS by:
 - 1) Verifying that on recirculation flow each motor-driven pump develops a differential pressure of greater than or equal to 1460 psid when tested pursuant to Specification 4.0.5;
 - 2) Verifying that on recirculation flow the steam turbine-driven pump develops a differential pressure of greater than or equal to 1640 psid when the secondary steam supply pressure is greater than 800 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.
- c. At least once per 18 months during shutdown by verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of an Auxiliary Feedwater Actuation test signal. For the steam turbine-driven auxiliary feedwater pump, the provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

4.7.1.2.2 An auxiliary feedwater flow path to each steam generator shall be demonstrated OPERABLE following each COLD SHUTDOWN of greater than 30 days prior to entering MODE 2 by verifying flow to each steam generator.