Docket Files

Docket No.: 50-423

JAN 22 1986

Mr. John F. Opeka
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P.O. Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT NO. 1 TO FACILITY OPERATING LICENSEE NO. NPF-44 - MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

The Commission has issued the enclosed Amendment No. 1 to Facility Operating License No. NPF-44 for the Millstone Nuclear Power Station, Unit No. 3. The amendment consists of revising Technical Specification Table 3.3-9, Remote Shutdown Instrumentation, by deleting the transfer switches associated with valves FWA*AOV-61A, 61B, 62A, and 62B (items 7, 8, 9, and 10 in the table). The change resolves an administrative error made in the compilation of Table 3.3-9 by making the table reflective of the actual plant configuration. The amendment is in response to your application, dated January 15, 1986.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration And Opportunity for Hearing will be included in the Commission's Monthly Notice.

Sincerely,

/S/

Vincent Noonan, Director
PWR Project Directorate #5
Division of PWR Licensing-A

Enclosures:
1. Amendment No. 1 to NPF-44
2. Safety Evaluation

cc: See next page

PWR-A
JLMilhoan
1/22/86

PWR-A/FOB
VBenaroya
1/27/86

Distribution:

Docket Files	OELD	M. Rushbrook
NRC PDR	E. Jordan	E. Doolittle
Local PDR	B. Grimes	ACRS (10)
PD#5 Reading File		

OFC	PD#5	PD#5	PD#5	DIR:PD#5			
NAME	MRushbrook:vt	EDoolittle	LOlshan	VNoonan			
DATE	1/23/86	1/22/86	1/22/86	1/22/86			

OFFICIAL RECORD COPY

2. Accordingly, Facility Operating License No. NPF-44 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

Revise paragraph 2C.(2) to read as following:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Vincent Noonan, Director
PWR Project Directorate #5
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: JAN 22 1986

PWR-A
JLMilhoan
1/22/86

PWR-A/FOB
VBenaroya
1/22/86

OFC	PD#5	PD#5	PD#5		PD#5		
NAME	MRushbrook:vt	EDoolittle	LOlshan	OELD	VNoonan		
DATE	1/ /86	1/22/86	1/22/86	1/ /86	1/ /86		

OFFICIAL RECORD COPY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JAN 22 1986

Docket No.: 50-423

Mr. John F. Opeka
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P.O. Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT NO. 1 TO FACILITY OPERATING LICENSEE NO. NPF-44 - MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

The Commission has issued the enclosed Amendment No. 1 to Facility Operating License No. NPF-44 for the Millstone Nuclear Power Station, Unit No. 3. The amendment consists of revising Technical Specification Table 3.3-9, Remote Shutdown Instrumentation, by deleting the transfer switches associated with valves FWA*AOV-61A, 61B, 62A, and 62B (items 7, 8, 9, and 10 in the table). The change resolves an administrative error made in the compilation of Table 3.3-9 by making the table reflective of the actual plant configuration. The amendment is in response to your application, dated January 15, 1986.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration And Opportunity for Hearing will be included in the Commission's Monthly Notice.

Sincerely,

Vincent Noonan, Director
PWR Project Directorate #5
Division of PWR Licensing-A

Enclosures:
1. Amendment No. 1 to NPF-44
2. Safety Evaluation

cc: See next page

Mr. J. F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit No. 3

cc:
Gerald Garfield, Esq.
Day, Berry & Howard
City Place
Hartford, Connecticut 06103-3499

Mr. Maurice R. Scully, Executive
Director
Connecticut Municipal Electric
Energy Cooperative
268 Thomas Road
Groton, Connecticut 06340

Robert W. Bishop, Esq.
Corporate Secretary
Northeast Utilities
Post Office Box 270
Hartford, Connecticut 06141

Mr. T. Rebelowski
Senior Resident Inspector Office
U. S. Nuclear Regulatory Commission
Millstone III
P. O. Box 615
Waterford, Connecticut 06385

Mr. Michael L. Jones, Manager
Project Management Department
Massachusetts Municipal Wholesale
Electric Company
Post Office Box 426
Ludlow, Massachusetts 01056

Regional Administrator
U. S. NRC, Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Mr. Karl Abraham
Public Affairs Office, Region I
U. S. Nuclear Regulatory Commission,
King of Prussia, Pennsylvania 19406

Ms. Jane Spector
Federal Energy Regulatory Commission
825 N. Capitol Street, NE
Room 8608C
Washington, D.C. 20426

Mr. Kevin McCarthy, Director
Radiation Control Unit
Department of Environmental Protection
State Office Building
Hartford, Connecticut 06115

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY,ET AL.*

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment, dated January 15, 1986, filed by the Northeast Nuclear Energy Company, as agent and representative of 15 utilities listed below and hereafter referred to as licensees, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I:

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

---

*Northeast Nuclear Energy Company is authorized to act as agent and representative for the following Owners: Central Maine Power Company, Central Vermont Public Service Corporation, Chicopee Municipal Lighting Plant, City of Burlington, Vermont, Connecticut Municipal Electric Energy Cooperative, The Connecticut Light and Power Company, Fitchburg Gas and Electric Light Company, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, Public Service Company of New Hampshire, The United Illuminating Company, The Village of Lyndonville Electric Department, Western Massachusetts Electric Company, and Vermont Electric Generation and Transmission Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, Facility Operating License No. NPF-44 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

Revise paragraph 2C.(2) to read as following:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Vincent Noonan, Director
PWR Project Directorate #5
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: JAN 22 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-44

DOCKET NO. 50-423

Revise "Appendix A" Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf page(*) has been provided to maintain document completeness.

REMOVE	INSERT
3/4 3-54	3/4 3-54
	3/4 3-53*
3/4 3-55	3/4 3-55
3/4 3-56	3/4 3-56

INSTRUMENTATION

REMOTE SHUTDOWN INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.5 The Remote Shutdown Instrumentation transfer switches, power, controls and monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

a. With the number of OPERABLE remote shutdown monitoring channels less than the Minimum Channels OPERABLE as required by Table 3.3-9, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in HOT SHUTDOWN within the next 12 hours.

b. With one or more Remote Shutdown Instrumentation transfer switches, power, or control circuits inoperable, restore the inoperable switch(s)/circuit(s) to OPERABLE status within 7 days, or be in HOT STANDBY within the next 12 hours.

c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.5.1 Each remote shutdown monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-6.

4.3.3.5.2 Each Remote Shutdown Instrumentation transfer switch, power and control circuit including the actuated components, shall be demonstrated OPERABLE at least once per 18 months.

TABLE 3.3-9

REMOTE SHUTDOWN INSTRUMENTATION

INSTRUMENT	TOTAL NO. READOUT LOCATION	MINIMUM OF CHANNELS	CHANNELS OPERABLE
1. Reactor Trip Breaker Indication	Reactor Trip Switchgear	1/trip breaker	1/trip breaker
2. Pressurizer Pressure	Aux. Shutdown Panel	2	1
3. Pressurizer Level	Aux. Shutdown Panel	2	1
4. Steam Generator Pressure	Aux. Shutdown Panel	2/steam generator	1/steam generator
5. Steam Generator Water Level	Aux. Shutdown Panel	2/steam generator	1/steam generator
6. Auxiliary Feedwater Flow Rate	Aux. Shutdown Panel	1/steam generator	1/steam generator
7. Loop Hot Leg Temperature	Aux. Shutdown Panel	1/loop	1/loop
8. Loop Cold Leg Temperature	Aux. Shutdown Panel	1/loop	1/loop
9. Reactor Coolant System Pressure (Wide Range)	Aux. Shutdown Panel	2	1
10. DWST Level	Aux. Shutdown Panel	2	1
11. RWST Level	Aux. Shutdown Panel	2	1
12. Containment Pressure	Aux. Shutdown Panel	2	1
13. Emergency Bus Voltmeters	Aux. Shutdown Panel	1/train	1/train
14. Source Range Count Rate	Aux. Shutdown Panel	2	1
15. Intermediate Range Flux	Aux. Shutdown Panel	2	1
16. Boric Acid Tank Level	Aux. Shutdown Panel	2/tank	1/tank

TRANSFER SWITCHES	SWITCH LOCATION
1. Auxiliary Feedwater Isolation FWA*MOV35A	Transfer Switch Panel
2. Auxiliary Feedwater Isolation FWA*MOV35B	Transfer Switch Panel
3. Auxiliary Feedwater Isolation FWA*MOV35C	Transfer Switch Panel
4. Auxiliary Feedwater Isolation FWA*MOV35D	Transfer Switch Panel
5. Auxiliary Feedwater Pump Ah. Suction FWA*AOV23A	Transfer Switch Panel
6. Auxiliary Feedwater Pump Ah. Suction FWA*AOV23B	Transfer Switch Panel

TABLE 3.3-9 (Continued)

REMOTE SHUTDOWN INSTRUMENTATION

TRANSFER SWITCHES	SWITCH LOCATION
7. Turbine Driven Pump Steam Supply MSS*AOV31A	Transfer Switch Panel
8. Turbine Driven Pump Steam Supply MSS*AOV31B	Transfer Switch Panel
9. Turbine Driven Pump Steam Supply MSS*AOV31D	Transfer Switch Panel
10. Reactor Vessel Head Vent Isolation RCS*SV8095A	Transfer Switch Panel
11. Reactor Vessel Head Vent Isolation RCS*SV8095B	Transfer Switch Panel
12. Reactor Vessel Head Vent Isolation RCS*SV8096A	Transfer Switch Panel
13. Reactor Vessel Head Vent Isolation RCS*SV8096B	Transfer Switch Panel
14. Reactor Vessel to Excess Letdown RCS*MV8098	Transfer Switch Panel
15. Pressurizer Level Control RCS*LCV459	Transfer Switch Panel
16. Pressurizer Level Control RCS*LCV460	Transfer Switch Panel
17. Letdown Orifice Isolation CHS*AV8149A	Transfer Switch Panel
18. Letdown Orifice Isolation CHS*AV8149B	Transfer Switch Panel
19. Letdown Orifice Isolation CHS*AV8149C	Transfer Switch Panel
20. Volume Control Tank Outlet Isolation CHS*LCV112B	Transfer Switch Panel
21. Volume Control Tank Outlet Isolation CHS*LCV112C	Transfer Switch Panel
22. RWST to CHS Pump Suction CHS*LCV112D	Transfer Switch Panel
23. RWST to CHS Pump Suction CHS*LCV112E	Transfer Switch Panel
24. Charging to RCS Isolation CHS*AV8146	Transfer Switch Panel
25. Charging to RCS Isolation CHS*AV8147	Transfer Switch Panel
26. Boric Acid Gravity Feed CHS*MV8507A	Transfer Switch Panel
27. Boric Acid Gravity Feed CHS*MV8507B	Transfer Switch Panel

TABLE 3.3-9 (Continued)

REMOTE SHUTDOWN INSTRUMENTATION

TRANSFER SWITCHES	SWITCH LOCATION
28. Charging Header Isolation Bypass CHS*MV8116	Transfer Switch Panel
29. Pressurizer Heater Backup RCS*H1A (Group A)	Transfer Switch Panel
30. Pressurizer Heater Backup RCS*H1B (Group B)	Transfer Switch Panel

CONTROL CIRCUITS	SWITCH LOCATION
1. Auxiliary Feedwater Flow Control FWA*HV31A	Auxiliary Shutdown Panel
2. Auxiliary Feedwater Flow Control FWA*HV31B	Auxiliary Shutdown Panel
3. Auxiliary Feedwater Flow Control FWA*HV31C	Auxiliary Shutdown Panel
4. Auxiliary Feedwater Flow Control FWA*HV31D	Auxiliary Shutdown Panel
5. Auxiliary Feedwater Flow Control FWA*HV32A	Auxiliary Shutdown Panel
6. Auxiliary Feedwater Flow Control FWA*HV32B	Auxiliary Shutdown Panel
7. Auxiliary Feedwater Flow Control FWA*HV32C	Auxiliary Shutdown Panel
8. Auxiliary Feedwater Flow Control FWA*HV32D	Auxiliary Shutdown Panel
9. Auxiliary Feedwater Flow Control FWA*HV36A	Auxiliary Shutdown Panel
10. Auxiliary Feedwater Flow Control FWA*HV36B	Auxiliary Shutdown Panel
11. Auxiliary Feedwater Flow Control FWA*HV36C	Auxiliary Shutdown Panel
12. Auxiliary Feedwater Flow Control FWA*HV36D	Auxiliary Shutdown Panel

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NPF-44

NORTHEAST NUCLEAR ENERGY COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

INDRODUCTION AND DISCUSSION

By letter dated January 15, 1986, the licensee proposed to change Technical Specification Table 3.3-9, Remote Shutdown Instrumentation, by deleting the transfer switches associated with auxiliary feedwater system valves 3FWA-AOV-61A, B and 62 A, B. The purpose of the change is to correct an error in Table 3.3-9; with the change, the table will reflect actual plant configuration.

During plant startup and normal plant shutdown, operation of the auxiliary feedwater (AFW) system involves using the safety-related, air operated valves in each motor driven AFW pump suction line (valves 61A, B). The suction valves 61A and 61B are normally open and receive a safety signal to open on AFW pump start, safety injection or loss of power to assure proper alignment with the safety grade demineralized water storage tank (DWST). The discharge line cross-connect valves 62A and 62B are normally closed, and receive a safety signal to close on AFW pump start, safety injection or loss of power to assure independence between the two motor driven pump trains during hot standby. This allows each motor-driven pump train to feed a pair of steam generators directly from the DWST. During normal plant startup and shutdown operations, these valves are repositioned to allow use of the nonsafety grade condensate water storage tank to preserve the water inventory in the safety grade DWST.

The full capacity turbine-driven AFW pump, which supplies water to all four steam generators from the DWST, contains only locked-open manual suction valves thereby assuring pump availability for all safe shutdown operations. In the event that the control room must be evacuated, the steam driven AFW pump is the primary means of removing decay heat. Control over this pump is from the auxiliary shutdown panel (ASP).

Technical Specification Table 3.3-9 and FSAR Table 7.4-1 currently indicate that valves 3FWA-AOV 61A, B and 62 A, B can be operated from the ASP. This, however, is not a prerequisite for the AFW system to satisfy safe shutdown and Appendix R requirements because of the capability at the ASP to control the steam-driven AFW pump. The applicant stated that the existing error in the Technical Specifications (and FSAR drawings) is due to a late system design change.

During development of the final design drawings, it was determined that control of the subject AFW system valves for the ASP was not necessary. However, through an oversight, a design drawing showing these switches removed from the ASP was not incorporated into the FSAR. As a result, the Technical Specifications were prepared under the supposition that the switches were located at the ASP.

EVALUATION

The proposed change to the Technical Specifications does not increase the probability of occurrence or the consequences of an accident previously evaluated, or create the possibility of a new or different kind of accident. The control over the four AFW system valves in question is not necessary from the remote shutdown panel to satisfy safe shutdown or Appendix R requirements since alternate safe shutdown capability is provided for at the ASP.

Therefore, the deletion of the four transfer switches associated with valves FWA-AOV-61A, B and 62 A, B from Table 3.3-9 is being done to correct an identified error in the Technical Specifications.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5 (d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with issuance of this amendment.

FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The state was informed by telephone on January 17, 1986 of our proposed no significant hazards consideration and had no comments. Based on our review of the licensee's submittal as described in our above evaluation and for the reasons stated below, we have made a final determination that the licensee's amendment request does not involve a significant hazards consideration.

The Commission has provided guidance for the application of the criteria in 10 CFR 50.92 by providing examples of amendments that are considered not likely to involve significant hazards considerations (48 FR 148 70); example (i) lists administrative changes. The application corrects an administrative error which allowed an erroneous FSAR change to be incorporated into the Technical Specifications. Therefore, the Commission has determined that the application does not involve a significant hazards consideration.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: JAN 2 2 1986

The following NRC personnel have contributed to this Safety Evaluation:

L. Olshan
E. Doolittle
R. Goel