

August 7, 1987

*DUR 016*

Docket No. 50-423

Mr. Edward J. Mrocza  
Senior Vice President  
Nuclear Engineering and Operations  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, CT 06141-0270

Dear Mr. Mrocza:

SUBJECT: ISSUANCE OF AMENDMENT (TAC #64771)

The Commission has issued the enclosed Amendment No. 8 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated February 26, 1987.

This amendment revises Millstone Unit No. 3 Technical Specifications Tables 3.3-1 and 3.3-2 by deleting the functional unit 21 (Reactor Trip Bypass Breakers).

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

Robert L. Ferguson, Project Manager  
Project Directorate I-4  
Division of Reactor Projects I/II

Enclosures:

- 1. Amendment No. 8 to NPF-49.
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. E. J. Mroczka  
Northeast Nuclear Energy Company

Millstone Nuclear Power Station  
Unit No. 3

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.\*

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8  
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated February 26, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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\*Northeast Nuclear Energy Company is authorized to act as agent and representative for the following Owners: Central Maine Power Company, Central Vermont Public Service Corporation, Chicopee Municipal Lighting Plant, City of Burlington, Vermont, Connecticut Municipal Electric Light Company, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, The Village of Lyndonville Electric Department, Western Massachusetts Electric Company, and Vermont Electric Generation and Transmission Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

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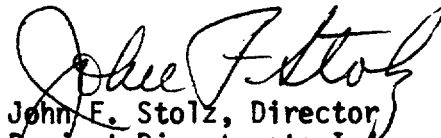
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 8, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 7, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 8

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove

3/4 3-4  
3/4 3-9

Insert

3/4 3-4  
3/4 3-9

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
12. Reactor Coolant Flow--Low					
a. Single Loop (Above P-8)	3/loop in each operating loop	2/loop in any operating loop	2/loop in each operating loop	1	6#
b. Two Loops (Above P-7 and below P-8)	3/loop in each operating loop	2/loop in two operating loops	2/loop each operating loop	1	6#
13. Steam Generator Water Level--Low-Low	4/stm. gen. in each operating stm. gen.	2/stm. gen. in any operating stm. gen.	3/stm. gen. each operating stm. gen.	1, 2	6# (1)
14. Low Shaft Speed--Reactor Coolant Pumps					
a. Four loop operation	4-1/pump	2	3	1**	6#
b. Three loop operation	3-1/pump	2	2	1**	6#
15. Turbine Trip					
a. Low Fluid Oil Pressure	3	2	2	1***	12#
b. Turbine Stop Valve Closure	4	4	4	1***	6#
16. Safety Injection Input from ESF	2	1	2	1, 2	10
17. Reactor Trip System Interlocks					
a. Intermediate Range Neutron Flux, P-6	2	1	2	2##	8
b. Low Power Reactor Trips Block, P-7					
P-10 Input	4	2	3	1	8
or					
P-13 Input	2	1	2	1	8

MILLSTONE - UNIT 3

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TABLE 3.3-1 (Continued)  
REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
17. Reactor Trip System Interlocks (Continued)					
c. Power Range Neutron Flux, P-8	4	2	3	1	8
d. Power Range Neutron Flux, P-9	4	2	3	1	8
e. Power Range Neutron Flux, P-10	4	2	3	1,2	8
18. Reactor Trip Breakers	2 2	1 1	2 2	1, 2 3*, 4*, 5*	10, 13 11
19. Automatic Trip and Interlock Logic	2 2	1 1	2 2	1, 2 3*, 4*, 5*	10 11
20. Three Loop Operation Bypass Circuitry	8 (1 switch per loop in each train)	2 (From different loop switches in bypass)	8	1, 2	1

TABLE 3.3-2 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION RESPONSE TIMES

<u>FUNCTIONAL UNIT</u>	<u>RESPONSE TIME</u>
12. Reactor Coolant Flow--Low	
a. Single Loop (Above P-8)	< 1 second
b. Two Loops (Above P-7 and below P-8)	< 1 second
13. Steam Generator Water Level--Low-Low	< 2 seconds
14. Low Shaft Speed-Reactor Coolant Pumps	< 0.6 second**
15. Turbine Trip	
a. Low Fluid Oil Pressure	N.A.
b. Turbine Stop Valve Closure	N.A.
16. Safety Injection Input from ESF	N.A.
17. Reactor Trip System Interlocks	N.A.
18. Reactor Trip Breakers	N.A.
19. Automatic Trip and Interlock Logic	N.A.
20. Three Loop Operation Bypass Circuitry	N.A.

\*\*Speed sensors are exempt from response time testing. Response time of the speed signal portion of the channel shall be measured from detector output or first electronic component in the channel.

MILLSTONE - UNIT 3

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Amendment No. 8



TABLE 4.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

MILLSTONE - UNIT 3

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<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Manual Reactor Trip	N.A.	N.A.	N.A.	R(14)	N.A.	1, 2, 3*, 4*, 5*
2. Power Range, Neutron Flux						
a. High Setpoint	S	D(2, 4), M(3, 4), Q(4, 6), R(4, 5)	Q(17)	N.A.	N.A.	1, 2
b. Low Setpoint	S	R(4)	S/U(1)	N.A.	N.A.	1***, 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R(4)	Q(17)	N.A.	N.A.	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R(4)	Q(17)	N.A.	N.A.	1, 2
5. Intermediate Range, Neutron Flux	S	R(4, 5)	S/U(1)	N.A.	N.A.	1***, 2
6. Source Range, Neutron Flux	S	R(4, 5)	S/U(1), Q(9,17)	N.A.	N.A.	2**, 3, 4, 5
7. Overtemperature $\Delta T$	S	R(12)	Q(17)	N.A.	N.A.	1, 2
8. Overpower $\Delta T$	S	R	Q(17)	N.A.	N.A.	1, 2
9. Pressurizer Pressure--Low	S	R	Q(17,18)	N.A.	N.A.	1
10. Pressurizer Pressure--High	S	R	Q(17,18)	N.A.	N.A.	1, 2
11. Pressurizer Water Level--High	S	R	Q(17)	N.A.	N.A.	1
12. Reactor Coolant Flow--Low	S	R	Q(17)	N.A.	N.A.	1



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NO. NPF-49  
NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.  
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3  
DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated February 26, 1987, the Northeast Nuclear Energy Company (licensee) requested a change to the Millstone Unit 3 Technical Specifications Tables 3.3-1 and 3.3-2. The proposed change would delete functional unit 21 (Reactor Trip Bypass Breakers) from these tables which impose limiting condition of operation and response time requirements.

This change was proposed in response to the staff's letter, dated November 19, 1986, which informed the licensee of an apparent discrepancy that they had noticed in the Millstone Unit No. 3 Technical Specifications as compared to Generic Letter 85-09 and to the Westinghouse Standard Technical Specifications. Specifically, the staff noted that the action statements 10 and 11 in Table 3.3-1 impose Limiting Condition of Operation requirements that are not appropriate for the reactor trip bypass breakers. By letter dated December 12, 1986, the licensee committed that the Millstone Unit No. 3 Technical Specification Table 3.3-1 will be revised to make it consistent with Generic Letter 85-09 and the Westinghouse Standard Technical Specifications.

2.0 EVALUATION

The reactor trip system consists of trains A and B operating the reactor trip breaker's undervoltage and shunt coils to open the trip breakers when a trip signal exists. The trains are independent and redundant. The reactor trip bypass breakers are normally open and shut one at a time, only for reactor trip breaker testing, Table 4.3-1 of the Millstone Unit No. 3 Technical Specifications requires that the bypass breakers be checked for tripability prior to use. None of this will change per this Technical Specification change. Table 3.3-1 imposes a limiting condition of operation for the reactor trip bypass breakers with inappropriate action statements. Table 3.3-2 does not impose a response time requirement for the reactor trip bypass breakers.

This proposed change to Table 3.3-1 will allow continued plant operation with one or two inoperable bypass breakers, as they will not affect the reactor trip system. The proposed change to Table 3.3-2 is editorial because a response time requirement was not previously imposed. The proposed change makes the requirements for the reactor trip bypass breakers consistent with Generic Letter 85-09 and the Westinghouse Standard Technical Specifications. We find these changes acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 7, 1987

Principal Contributors:

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