

July 9, 1990

Docket No. 50-423

Mr. Edward J. Mroczka  
Senior Vice President  
Nuclear Engineering and Operations  
Connecticut Yankee Atomic Power Company  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: MILLSTONE UNIT 3 - ISSUANCE OF AMENDMENT (TAC NO. 76490)

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated March 30, 1990.

The amendment changes the values in Technical Specification (TS) Table 3.3-4, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints", associated with the steam generator water level setpoint.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

David H. Jaffe, Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 52 to NPF-49
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52  
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated March 30, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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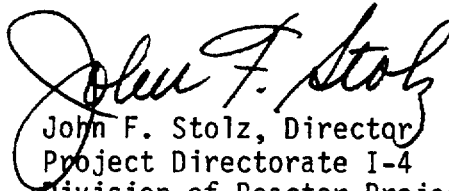
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 52, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 9, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 52

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace page 3/4 3-28 of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contain vertical lines indicating the areas of change.

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA) Z</u>		<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. Turbine Trip and Feedwater Isolation					
a. Automatic Actuation Logic Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A.
b. Steam Generator Water Level--High-High (P-14)	5.25	3.76	1.50	≤ 80.45% of narrow range instrument span.	≤ 81.47% of narrow range instrument span.
c. Safety Injection Actuation Logic	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.				
d. T <sub>ave</sub> Low Coincident with Reactor Trip (P-4)					
1) Four Loops Operating	N.A.	N.A.	N.A.	≥ 564°F	≥ 560.6°F
2) Three Loops Operating	N.A.	N.A.	N.A.	≥ 564°F	≥ 560.6°F
6. Auxiliary Feedwater					
a. Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A.
c. Steam Generator Water Level--Low-Low					
1) Start Motor-Driven Pumps	18.10	16.64	1.50	≥ 18.10% of narrow range instrument span.	≥ 17.11% of narrow range instrument span.

MILLSTONE - UNIT 3

3/4 3-28

Amendment No. 12, 31 52



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 52

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

INTRODUCTION

By application for license amendment dated March 30, 1990, Northeast Nuclear Energy Company, et al. (the licensee), requested changes to Millstone Unit 3 Technical Specifications (TS).

The proposed amendment would change the values in Technical Specification (TS) Table 3.3-4, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints", associated with the steam generator water level setpoint.

DISCUSSION AND EVALUATION

The Millstone Unit 3 Engineered Safety Features Actuation System (ESFAS) is provided with a "high-high" steam generator level setpoint (SG high-high) that, when actuated, provides a turbine trip and feedwater isolation. The purpose of the SG high-high trip is to prevent overfilling of the steam generators.

The proposed revised setpoint will lower the SG level (narrow range) at which a turbine trip and feedwater isolation will occur. The current trip setpoint is at 82% narrow range level. The proposed revised trip setpoint is 80.45% level. The proposed changes are necessary due to an increase in revised channel statistical allowances for abnormal environmental conditions. The associated values for total allowance (TA), statistical sum of errors excluding sensor and instrument rack errors (Z), sensor error (S), and allowable value for turbine trip and feedwater isolation on high-high SG water level would be changed accordingly.

The SG high-high trip is credited in terminating the excess feedwater event as described in Section 15.1.2, "Feedwater System Malfunctions that Result in an Increase in Feedwater Flow" of the Millstone Unit 3 FSAR. The revised setpoint assures that an isolation signal will be generated as assumed in the analysis.

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The revised SG high-high trip is calculated using the currently approved Millstone Unit 3 (Westinghouse) setpoint methodology as described in WCAP-10991, "Westinghouse Setpoint Methodology for Protection Systems, Millstone Unit No. 3" which is also used to calculate the other trip setpoints in TS Table 3.3-4. Since the change in trip setpoint is minor, the projected course of the excess feedwater event will not change. The proposed change will enhance the safety performance of the ESFAS since the feedwater isolation will be initiated sooner.

Based upon the above, we conclude that the proposed changes to TS Table 3.3-4 are acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 9, 1990

Principal Contributor: D. Jaffe