

March 17, 1995

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, CT 06141-0270

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M90492)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 106 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated August 4, 1994.

The amendment relaxes the setpoint tolerance for the pressurizer safety valves and the main steam safety valves from $\pm 1\%$ to $\pm 3\%$.

A copy of the related Safety Evaluation is also enclosed. Also enclosed is the Notice of Issuance which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by Maudette Griggs

for: Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 106 to NPF-49
2. Safety Evaluation
3. Notice

cc w/encls: See next page

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OFFICE	LA:PDI-4	PM:PD144	D:PD144	OGC
NAME	SNorris	VRooney:bf	PMcKee	S. H. W. W. W. W.
DATE	02/9/95	02/9/95	02/9/95	02/9/95

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DOCUMENT NAME: G:\ROONEY\M90492.AMD

OFFICE	LA:PDI-4	PM:PDI-4	D:PDI-4	OGC	
NAME	SNorris	VRooney:bf	PMcKee	Sutton	Michael
DATE	02/9/95	02/9/95	02/9/95	02/9/95	02/ /95

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 17, 1995

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Sincerely,

A handwritten signature in cursive script, reading "Maudette Griggs for", is written over the typed name.

Vernon L. Rooney, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-423

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cc w/encls: See next page

Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated August 4, 1994 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

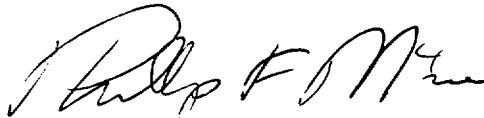
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 106 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 17, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 4-9
3/4 4-10
3/4 7-3

Insert

3/4 4-9
3/4 4-10
3/4 7-3

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY VALVES

SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.2.1 A minimum of one pressurizer Code safety valve shall be OPERABLE with a lift setting* of 2500 psia \pm 3%.**

APPLICABILITY: MODE 4.

ACTION:

With no pressurizer Code safety valve OPERABLE, immediately suspend all operations involving positive reactivity changes and place an OPERABLE RHR loop into operation in the shutdown cooling mode.

SURVEILLANCE REQUIREMENTS

4.4.2.1 No additional Surveillance Requirements other than those required by Specification 4.0.5.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

**The lift setting shall be within \pm 1% following pressurizer Code safety valve testing.

REACTOR COOLANT SYSTEM

OPERATING

LIMITING CONDITION FOR OPERATION

3.4.2.2 All pressurizer Code safety valves shall be OPERABLE with a lift setting* of 2500 psia \pm 3%.**

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

With one pressurizer Code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.2.2 No additional Surveillance Requirements other than those required by Specification 4.0.5.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

**The lift setting shall be within \pm 1% following pressurizer Code safety valve testing.

TABLE 3.7-3

STEAM LINE SAFETY VALVES PER LOOP

<u>VALVE NUMBER</u>	<u>LIFT SETTING* ($\pm 3\%$)**</u>	<u>ORIFICE SIZE</u>
<u>LOOP 1</u>		
RV22A	1185 psig	16.0 square inches
RV23A	1195 psig	16.0 square inches
RV24A	1205 psig	16.0 square inches
RV25A	1215 psig	16.0 square inches
RV26A	1225 psig	16.0 square inches
<u>LOOP 2</u>		
RV22B	1185 psig	16.0 square inches
RV23B	1195 psig	16.0 square inches
RV24B	1205 psig	16.0 square inches
RV25B	1215 psig	16.0 square inches
RV26B	1225 psig	16.0 square inches
<u>LOOP 3</u>		
RV22C	1185 psig	16.0 square inches
RV23C	1195 psig	16.0 square inches
RV24C	1205 psig	16.0 square inches
RV25C	1215 psig	16.0 square inches
RV26C	1225 psig	16.0 square inches
<u>LOOP 4</u>		
RV22D	1185 psig	16.0 square inches
RV23D	1195 psig	16.0 square inches
RV24D	1205 psig	16.0 square inches
RV25D	1215 psig	16.0 square inches
RV26D	1225 psig	16.0 square inches

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

**The lift setting shall be within $\pm 1\%$ following main steam line Code safety valve testing.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 106

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated August 4, 1994, the Northeast Nuclear Energy Company (NNECO/the licensee) submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 Technical Specifications (TS). The requested changes would change TSs 3.4.2.1 and 3.4.2.2 and Table 3.7-3, and involve relaxing the setpoint tolerance for the pressurizer safety valves (PSVs) and the main steam safety valves (MSSVs) from $\pm 1\%$ to $\pm 3\%$. The requested changes are supported by the licensee's Cycle 4 Reload analysis.

2.0 EVALUATION

Overpressure protection for the reactor coolant system (RCS) and the main steam system is provided, in part, by the PSVs and the MSSVs located on the pressurizer and on the four main steam lines, respectively. The current TS requires one PSV to be operable in Mode 4 and all PSVs are required operable in Modes 1, 2, and 3 with a lift setting of 2500 psia $\pm 1\%$. All of the MSSVs are required operable during Modes 1, 2 and 3 with lift settings in increments of 10 psig, ranging from 1185 psig to 1225 psig, $\pm 1\%$.

By letter dated November 1, 1990, NNECO submitted proposed TS changes and a reload analysis specific to the Millstone, Unit 3 Cycle 4 Reload. The reload analysis incorporated the VANTAGE 5 Hybrid (5H) fuel design and also examined for the the impact on the safety analysis of the increase in the setpoint tolerance for the PSVs and MSSVs from $\pm 1\%$ to $\pm 3\%$. The TS changes proposed by the current submittal are supported by the licensee's Cycle 4 Reload analysis, which was approved by the staff on March 11, 1991.

The November 1990 analysis for the non-loss of coolant accident (non-LOCA) transients included revisions in the modeling of the PSVs and MSSVs. The licensee included the following in the analysis: (1) a 3% accumulation and a 3% setpoint drift; (2) the pressurizer pressure was allowed to reach full rated flow over the range of 2575 to 2652 psia; and (3) the steam generator pressure was allowed to reach 1320 psia - 110% of the steam generator design pressure. The licensee also indicated that the LOCA analyses considered the 3% MSSV drift from the setpoint for both the large and small break LOCA analyses, and the analyses remained bounding.

To support a future request to relax the setpoint of the PSVs and MSSVs from $\pm 1\%$ to $\pm 3\%$, the licensee reanalyzed the increase in the PSV and MSSV setpoint tolerance in their November 1990, Cycle 4 Reload analysis. The August 4, 1994, submittal is the request to change the setpoint tolerances based on the November 1990 analysis. The reload analysis was approved by the staff in License Amendment No. 60, issued March 11, 1991 and the results of the analysis still remain applicable. Therefore the staff approves the proposed TS changes.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published in the Federal Register on March 13, 1995 (60 FR 13476). Accordingly, based upon the environmental assessment, the staff has determined that the issuance of the amendment will not have a significant effect on the quality of the human environment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Brewer

Date: March 17, 1995

UNITED STATES NUCLEAR REGULATORY COMMISSIONNORTHEAST NUCLEAR ENERGY COMPANYDOCKET NO. 50-423NOTICE OF ISSUANCE OF AMENDMENT TOFACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 106 to Facility Operating License No. NPF-49 issued to Northeast Nuclear Energy Company (the licensee), which revised the Technical Specifications (TS) for operation of the Millstone Nuclear Power Station, Unit No. 3 located in New London County, Connecticut. The amendment is effective as of the date of issuance.

The amendment relaxes the setpoint tolerance for the pressurizer safety valves and the main steam safety valves from $\pm 1\%$ to $\pm 3\%$.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on October 12, 1994 (59 FR 51612). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (60 FR 13476).

For further details with respect to the action see (1) the application for amendment dated August 4, 1994, (2) Amendment No. 106 to License No. NPF-49, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Learning Resource Center, Three Rivers Community-Technical College, Thames Valley Campus, 574 New London Turnpike, Norwich, CT 06360.

Dated at Rockville, Maryland, this 17th day of March 1995.

FOR THE NUCLEAR REGULATORY COMMISSION



Vernon L. Rooney, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation