September 26, 1988

Docket No. 50-423

DISTRIBUTION
See attached page

Mr. Edward J. Mroczka
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 68358)

The Commission has issued the enclosed Amendment No. 23 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated May 19, 1988.

The amendment changes Technical Specification Section 4.6.1.3.a "Containment Air Locks" to allow the use of alternate methods for the leak rate testing of the containment air locks.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

original signed by

David H. Jaffe, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 23to NPF-49
2. Safety Evaluation

cc w/enclosures:
See next page

LA:PDI-4	PM:PDI-4	D:PDI-4	SPLB:NRR	OGC
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09/6/88	09/6/88	for 09/15/88	9/15/88	9/20/88

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AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. NPF-49

Docket File
NRC & Local PDRs
PDI-4 Reading
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cc: Plant Service list

DF01
1/1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.*

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 23
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated May 19, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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*Northeast Nuclear Energy Company is authorized to act as agent and representative for the following Owners: Central Maine Power Company, Central Vermont Public Service Corporation, Chicopee Municipal Lighting Plant, City of Burlington, Vermont, Connecticut Municipal Electric Light Company, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, The Village of Lyndonville Electric Department, Western Massachusetts Electric Company, and Vermont Electric Generation and Transmission Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF 49 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 23 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ronald W. Hernan for

John F. Stolz, Director
Project Directorate I 4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 26, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 23

FACILTIY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change. The corresponding overleaf page is provided to maintain document completeness.

Remove	Insert
3/4 6-6	3/4 6-6

TABLE 3.6-1

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

PENETRATION NO.	SYSTEM	RELEASE LOCATION
14	Normal Sump	Unit 2 Stack via Aerated Waste Drain Tank Vent.
67	Refueling Water Purification	Unit 2 Stack via Auxiliary Building Ventilation System above Spent Fuel Pool.
68	Refueling Water Purification	Unit 2 Stack via Auxiliary Building Ventilation System above Spent Fuel Pool.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

a. 1) Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying no detectable seal leakage by pressure decay when the volume between the door seals is pressurized to greater than or equal to P_a, 54.1 psia (39.4 psig), for at least 15 minutes;

or

2) Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying that the seal leakage is less than 0.01 L_a as determined by precision flow measurements when measured for at least 30 seconds with the volume between the seals at a constant pressure of greater than or equal to P_a, 54.1 psia (39.4 psig);

or

3) Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by completing an overall air lock leakage test per 4.6.1.3.b.

b. By conducting overall air lock leakage tests at not less than P_a, 54.1 psia (39.4 psig), and verifying the overall air lock leakage rate is within its limit:

1) At least once per 6 months,* and

2) Prior to establishing CONTAINMENT INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.**

c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

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*The provisions of Specification 4.0.2 are not applicable.
**This represents an exemption to Appendix J, paragraph III.D.2.(b)(ii), of 10 CFR Part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 23

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

INTRODUCTION

By application dated May 19, 1988, Northeast Nuclear Energy Company (NNECO) requested changes to the Technical Specifications (TS) for Millstone Unit 3. The proposed changes would modify TS 4.6.1.3.a, "Containment Air Locks" to allow the use of alternate test methods for the leak rate testing of the containment air locks.

DISCUSSION AND EVALUATION

At the present time, TS 4.6.1.3.a requires that the containment air locks be leak rate tested using the pressure decay method. The subject test must be conducted within 72 hours following each closing except when the air lock is used for multiple entries, then at least once per 72 hours. NNECO has proposed that two alternate containment air lock leak rate test methods should also be permitted. The alternate test methods would be the precision flow method and overall air lock leakage method which would be designated as TS 4.6.1.3.a.2 and 4.6.1.3.a.3, respectively.

Methods for determining containment leakage are specified in 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors." Specifically, Section III.B of Appendix J describes three test methods which are equally acceptable for the leak rate testing of containment air locks. The presently approved method, specified in existing TS 4.6.1.3.a.1 involves ". . . verifying no detectable seal leakage by pressure decay when the volume between the door seals is pressurized to greater than or equal to Pa, 54.1 psia (39.4 psig), for at least 15 minutes." The "pressure decay" test method is specifically permitted by Appendix J, Section III.B.1.(b) which describes such tests as follows:

> "(b) Measurement of the rate of pressure loss of the test chamber of the containment penetration pressurized with air, nitrogen, or pneumatic fluids specified in the technical specifications or associated bases."

NNECO has proposed new TS 4.6.1.3.a.2 to provide an alternate means of containment air lock testing to allow, ". . . verifying that the seal leakage is less than .01 L_a as determined by precision flow measurements when measured

for at least 30 seconds with the volume between the seals at a constant pressure of greater than or equal to Pa, 54.1 psia (39.4 psig)." The "precision test" method is also permitted by Appendix J, specifically, Section III.B.1.(c) as follows:

> "(c) Leakage surveillance by means of a permanently installed system with provisions for continuous or intermittent pressurization of individual or groups of containment penetrations and measurement of rate of pressure loss of air, nitrogen, or pneumatic fluid specified in the technical specification or associated bases through the leak paths."

Finally, NNECO has proposed a second alternate containment air lock test method to be specified in new TS 4.6.1.3.a.3 by referencing TS 4.6.1.3.b, as follows: ". . . conducting overall air lock leakage tests at not less than Pa, 54.1 psia (39.4 psig), and verifying the overall air lock leakage rate is within its limits." The second alternate test method, also permitted by Appendix J, Section III.B.1.(c) involves pressurizing the air lock itself and has the additional benefit of also, routinely, determining the leakage due to air lock penetrations.

Based upon the above, we conclude that the existing and two proposed alternate routine containment air lock test methods are permitted by Appendix J. Moreover, these test methods represent suitable means for determining containment air lock leakage at Millstone Unit 3. Accordingly, the proposed changes to Millstone Unit 3, TS 4.6.1.3.a are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 26, 1988

Principal Contributor:

D. H. Jaffe