Mr. John F. Opeka
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Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
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SUBJECT:

ISSUANCE OF AMENDMENT (TAC NO. M85726)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 108 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated January 26, 1993, with clarifying information submitted August 4, 1993.

The amendment changes the Technical Specifications governing electrical power systems, AC and DC power sources, and onsite power distribution for shutdown conditions (modes 5 and 6).

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly <u>Federal</u> <u>Register</u> notice.

Sincerely,

Original signed by

Vernon L. Rooney, Senior Project Manager Project Directorate I-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 108 to NPF-49

2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001April 12, 1995

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Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
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Vernon L. Roone, Senior Project Manager

Project Directorate I-3

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108 License No. NPF-49

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated January 26, 1993, as supplemented by letter dated August 4, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations:
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 108, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Phillip F. McKee, Director

Project Directorate I-3

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical

Specifications

Date of Issuance: April 12, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	<u>Insert</u>
3/4 8-10	3/4 8-10
3/4 8-15	3/4 8-15
3/4 8-18	3/4 8-18

A. C. SOURCES

SHUTDOWN

LIMITING CONDITION FOR OPERATION

- 3.8.1.2 As a minimum, the following A. C. electrical power sources shall be OPERABLE:
 - a. One circuit between the offsite transmission network and the Onsité Class 1E Distribution System, and
 - b. One diesel generator with:
 - 1) A day tank containing a minimum volume of 278 gallons of fuel.
 - 2) A fuel storage system containing a minimum volume of 32,760 gallons of fuel,
 - 3) A fuel transfer pump,
 - 4) Lubricating oil storage containing a minimum total volume of 280 gallons of lubricating oil, and
 - 5) Capability to transfer lubricating oil from storage to the diesel generator unit.

APPLICABILITY: MODES 5 and 6.

ACTION:

With less than the above minimum required A. C. electrical power sources OPERABLE, immediately suspend all operations involving CORE ALTERATIONS, positive reactivity changes, movement of irradiated fuel, crane operation with loads over the fuel storage pool, or operation with a potential for draining the reactor vessel; initiate corrective action to restore the required sources to OPERABLE status as soon as possible.

SURVEILLANCE REQUIREMENT

4.8.1.2 The above required A.C. electrical power sources shall be demonstrated OPERABLE by the performance of each of the requirements of Specifications 4.8.1.1.1, 4.8.1.1.2 (except for Specification 4.8.1.1.2a.6), and 4.8.1.1.3.

ELECTRICAL POWER SYSTEMS

D. C. SOURCES

SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.2.2 As a minimum, one 125-volt battery bank and its associated full-capacity charger shall be OPERABLE.

APPLICABILITY: MODES 5 and 6.

ACTION:

With the required battery bank and/or full-capacity charger inoperable, immediately suspend all operations involving CORE ALTERATIONS, positive reactivity changes, movement of irradiated fuel; crane operation with loads over the fuel storage pool, or operation with a potential for draining the reactor vessel; initiate corrective action to restore the required battery bank and full-capacity charger to OPERABLE status as soon as possible.

SURVEILLANCE REQUIREMENTS

4.8.2.2 The above required 125-volt battery bank and full-capacity charger shall be demonstrated OPERABLE in accordance with Specification 4.8.2.1.

ELECTRICAL POWER SYSTEMS

ONSTITE POWER DISTRIBUTION

SHUTDOWN

LIMITING CONDITION FOR OPERATION

- 3.8.3.2 As a minimum, the following electrical busses shall be energized in the specified manner:
 - a. One train of A.C. emergency busses consisting of one 4160-volt and four 480-volt A.C. emergency busses.
 - b. Two 120-volt A.C. vital busses energized from their associated inverters connected to their respective D.C. busses, and
 - c. Two 125-volt D.C. busses energized from their associated battery banks.

APPLICABILITY: MODES 5 and 6.

ACTION:

With any of the above required electrical busses not energized in the required manner, immediately suspend all operations involving CORE ALTERATIONS, positive reactivity changes, movement of irradiated fuel, crane operation with loads over the fuel storage pool, or operations with a potential for draining the reactor vessel, initiate corrective action to energize the required electrical busses in the specified manner as soon as possible.

SURVEILLANCE REQUIREMENTS

4.8.3.2 The specified busses shall be determined energized in the required manner at least once per 7 days by verifying correct breaker alignment and indicated voltage on the busses.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 108

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated January 26, 1993, as supplemented August 4, 1993, the Northeast Nuclear Energy Company (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 Technical Specifications (TSs). The requested changes would change the requirements for onsite power distribution during shutdown in TSs 3.8.1.2, 3.8.2.2, and 3.8.3.2. The August 4, 1993, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The licensee has proposed changes to TSs 3.8.1.2, 3.8.2.2, and 3.8.3.2 which affect the likelihood of an accident when there is insufficient AC, DC, or onsite power distribution. The licensee proposed to add a requirement to the Action Statements in TSs 3.8.2.2 and 3.8.3.2 to suspend operation of the crane over the fuel storage pool, when the crane is loaded. The licensee also proposed to expand the requirement in TS 3.8.1.2, to restore the required AC power as soon as possible, to include all of Modes 5 and 6. Currently, the licensee is required to restore the required AC power when low water level is present in Mode 6 or the loops not filled in Mode 5. Furthermore, the licensee proposed to change the Action Statement for TSs 3.8.1.2, 3.8.2.2, and 3.8.3.2 to suspend any operations with a potential for draining the reactor vessel. These changes proposed by the licensee are acceptable by the staff because they (1) do not effect the accident analyses for Modes 5 and 6 - fuel handling accident, boron dilution, core misload and cask drop, and (2) impose a greater restriction on plant operation.

To ensure that the reactor coolant system (RCS) is protected from pressure transients which exceed the limits of Appendix G, TS 3.4.9.3, "Overpressure Protection System," requires that two power operated relief valves or two residual heat removal suction relief valves or one of each or RCS vent opening of at least 5.4 square inches be operable. The licensee also requested to remove the requirement to depressurize and vent the RCS from TSs 3.8.1.2,

3.8.2.2., and 3.8.3.2, indicating that this requirement is already covered in TS 3.4.9.3, and is, therefore, a redundant requirement. The staff agrees that, since TS 3.4.9.3 is applicable to Modes 3, 4, 5, and 6 and adequately prevents overpressurization during shutdown conditions, the request is acceptable.

The staff has reviewed the licensee's submittals and determined that since the changes (1) do not effect the accident analyses, (2) impose a greater restriction on plant operation, or (3) are covered under another TS, the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 28058). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Brewer

Date: April 12, 1995