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Docket No. 50-416

Mr. Oliver D. Kingsley, Jr.
Vice President - Nuclear Operations
Mississippi Power & Light Company
P. O. Box 23054
Jackson, Mississippi 39205

Dear Mr. Kingsley:

SUBJECT: FEDERAL REGISTER NOTICE

RE: Grand Gulf Nuclear Station, Unit 1

Enclosed is an Individual Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing. This amendment was requested in Item 13 of the attachment to your letter dated August 12, 1985, as amended September 25, 1985 and supplemented October 5 and October 22, 1985 and May 30, 1986. This Notice was forwarded to the Office of the Federal Register and was published on September 4, 1986. A correction to the Notice, published in the Federal Register (51 FR 32979) on September 17, 1986, is also enclosed.

Sincerely,

Original Signed by

Lester L. Kintner, Project Manager
BWR Project Directorate No. 4
Division of BWR Licensing

Enclosures:
As Stated

cc w/enclosures:
See Next Page

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Mr. Oliver D. Kingsley, Jr.
Mississippi Power & Light Company

Grand Gulf Nuclear Station

cc:

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Wise, Carter, Child, Steen and Caraway
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The Honorable William J. Guste, Jr.
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Mississippi Power & Light Company
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UNITED STATES NUCLEAR REGULATORY COMMISSION

MISSISSIPPI POWER & LIGHT COMPANY

MIDDLE SOUTH ENERGY, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

DOCKET NO. 50-416

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS

CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-29 issued to Mississippi Power & Light Company, Middle South Energy, Inc., South Mississippi Electric Power Association, for operation of the Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi.

The proposed amendment would make the following changes in the Technical Specifications; add specifications in Table 3.3.3-1, "Emergency Core Cooling System (ECCS) Actuation Instrumentation" and Table 3.3.3-2, "Emergency Core Cooling System Actuation Instrumentation Setpoints" to incorporate interlock instrumentation which is designed to prevent inadvertent overpressurization of low design pressure emergency core cooling systems by the reactor coolant systems, and make associated changes in Table 3.3.3-3, "ECCS Response Times" and Surveillance Requirement 4.5.1 regarding response times of ECCS injection systems, Table 4.3.3.1-1, "ECCS Actuation Instrumentation Surveillance Requirements," Surveillance Requirement 4.4.3.2.2, "Reactor Coolant System Operational

Leakage," Table 3.4.3.2-2, "Reactor Coolant System Interface Valves Pressure Monitors Alarm," and Table 3.4.3.2-3 "Reactor Coolant System Interface Valves Pressure Interlocks." These proposed changes were requested in Item 13 of the attachment to the licensee's letter dated August 12, 1985, as amended September 25, 1985 and supplemented October 5 and October 22, 1985 and May 30, 1986. The changes requested in Item 12 of the August 12, 1985 letter were previously noticed and issued as Amendment No. 7 to GGNS Unit 1 License No. NPF-29 on November 8, 1985.

This notice supersedes a previous notice published in the Federal Register on August 28, 1985 (50 FR 34994). The previous notice was based on the licensee's initial application for amendment dated August 12, 1985. During its safety review of proposed changes to Technical Specifications for the ECCS injection valve interlocks the staff noted the licensee's proposed deletion of tests of response times for starting the ECCS systems associated with the injection valves, because the system response with valve interlocks would vary, depending on the rate of depressurization during a loss of coolant accident. The presently specified system response time (40 seconds) includes 10 seconds for starting an emergency diesel generator and 30 seconds for opening the injection valve in the system. In response to staff questions, regarding surveillance tests of injection valve opening, the licensee proposed by letter dated September 25, 1985 to include surveillance tests of the time for injection valves to move from the closed position to the open position (29 seconds). Surveillance tests of emergency diesel generator starting times (10 seconds) are presently included in Technical Specification 4.8.1.1.2. This notice is based on the revised application from that initially noticed which results in greater assurance that the ECCS injection valves will open within the design time. Appropriate changes

to the initial notice regarding ECCS injection valve response time have been incorporated in this notice.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has made an analysis of significant hazards considerations using the standards of 10 CFR 50.92 and has concluded that the proposed amendment does not involve a significant hazards consideration. The NRC staff has made a preliminary review of licensee's analysis and associated bases. Staff's discussion of the proposed amendment as it relates to the three standards follows.

The addition to the Technical Specifications of interlock instrumentation on pressure isolation valves, is needed to implement a design change required by a license condition. The present compensatory requirement for leak tests of low pressure core spray (LPCS) and low pressure coolant injection (LPCI) check valves would be deleted. The design change would result in an increase of 51°F in calculated peak cladding temperature to 2149°F during a postulated loss of coolant accident because of a longer time required for LPCS and LPCI injection valves to start to open. The required response time for LPCS

and LPCI injection valves to move from the closed to the open position (29 seconds) will be slightly faster than the required response time defined in present Technical Specifications (30 seconds). The calculated peak cladding temperature of 2149°F is still below the limiting 2200°F required by 10 CFR 50.46, so the safety margin is not affected. The design change will be performed in accordance with appropriate regulatory and industry codes and standards, the GGNS quality assurance program, and applicable requirements of the GGNS FSAR. Therefore, the design change would be consistent with the licensing basis. Because these changes will add requirements not presently included in the Technical Specifications which more than offset the removal of the compensatory leak test requirement, and because the change would result in the performance of the ECCS safety function without affecting the safety margin, this change does not significantly increase the probability or consequences of an accident previously evaluated or create the possibility of a new or different kind of accident from any accident previously evaluated, nor does it involve a significant reduction in a margin of safety.

Accordingly, the Commission proposes to determine that these changes do not involve a significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments should be addressed to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Copies of comments received may be examined at the NRC Public Document Room, 1717 H Street, NW, Washington, D.C.

By October 7, 1986 , the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may

be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Att: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, NW Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to Walter R. Butler, Director, BWR Project Directorate No. 4, Division of BWR Licensing: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page

number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Nicholas S. Reynolds, Esquire, Bishop, Lieberman, Cook, Purcell, and Reynolds, 1200 17th Street, N.W., Washington, D.C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board, that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated August 12, 1985 as amended September 25, 1985 and supplemented October 5 and October 22, 1985 and May 30, 1986 which is available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D.C., and at the Hinds Junior College, McLendon Library, Raymond, Mississippi 39154.

Dated at Bethesda, Maryland, this 28th day of August 1986.

FOR THE NUCLEAR REGULATORY COMMISSION


Walter R. Butler, Director
BWR Project Directorate No. 4
Division of BWR Licensing

—Significant rerouting of power cabling and associated conduits, ducts and supports.

—Increased surveillance on new or extended fire suppression and fire detection systems.

—Increased congestion in numerous plant locations complicating future plant modifications/operations.

The licensee stated that these costs are significantly in excess of those required to meet the underlying purpose of the rule. The staff concludes that "special circumstances" exist for the licensee's requested exemptions in that application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of Appendix R to 10 CFR Part 50. (see 10 CFR 50.12(a)(2)(ii)).

IV.

Accordingly, the Commission has determined pursuant to 10 CFR Part 50.12(a), that these twenty-seven technical exemptions discussed in Section III are authorized by law and will not endanger life or property or the common defense and security, and is otherwise in the public interest. The Commission hereby approves the twenty-seven requested exemptions from Appendix R of 10 CFR 50 section III.G as specifically identified in the Safety Evaluation dated September 10, 1986, which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, DC, and at the Local Public Document Room, located at the George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama.

Pursuant to 10 CFR 51.32, the Commission has determined that the issuance of the Exemption will have no significant impact on the environment (51 FR 32151, dated September 9, 1986).

This exemption is effective upon issuance.

Dated at Bethesda, Maryland this 10th day of September, 1986.

For the Nuclear Regulatory Commission.

Thomas M. Novak,

Acting Director, Division of PWR Licensing-A, Office of Nuclear Reactor Regulation.

[FR Doc. 86-21062 Filed 9-16-86; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-322-OL-5 (EP Exercise); (ASLBP No. 86-533-01-OL)]

Long Island Lighting Co. (Shore Nuclear Power Station, Unit 1); Schedule for Prehearing Conference and Making Limited Appearances

Before Administrative Judges Morton B. Margulies, Chairman, Dr. Jerry R. Kline, and Frederick J. Shon.

September 10, 1986.

Pursuant to the Board's Order of September 9, 1986 providing for the scheduling of a prehearing conference in conjunction with the taking of limited appearances we set the following schedule for the session. The schedule has been affected by the availability of adequate hearing facilities.

Limited appearances will be taken on September 23, 1986, at Hauppauge, New York, in the William H. Rogers Building, Suffolk County Center, Veterans Memorial Highway, between the hours of 9:30 a.m. to 12:00 noon, 1:30 p.m. to 4:00 p.m., and 6:00 p.m. to 9:00 p.m.

Limited appearances will be taken on September 25, 1986, at Riverhead, New York, in the Riverhead Town Hall, 200 Howell Avenue, between the hours of 9:30 a.m. to 12:00 noon, 1:30 p.m., to 4:00 p.m., and 6:00 p.m. to 9:00 p.m.

Limited appearances will be taken on September 26, 1986, at Mineola, New York, in the Executive Building, 1 West Street, between the hours of 9:30 a.m. to 12:00 noon, 1:30 to 4:00 p.m., and 6:00 p.m. to 9:00 p.m.

The prehearing conference, concerned with legal issues, in which only the parties are to participate, will be held on September 24, 1986, at Hauppauge, New York, in the New York State Court of Claims, State Office Building, Room 3B44, Veterans Memorial Highway, commencing at 9:30 a.m. The public is invited to attend the prehearing conference.

It is so Ordered.

Dated at Bethesda, Maryland, this 10th day of September, 1986.

The Atomic Safety and Licensing Board.

Morton B. Margulies,

Chairman, Administrative Law Judge.

Dr. Jerry R. Kline,

Administrative Judge.

Frederick J. Shon,

Administrative Judge.

[FR Doc. 86-21063 Filed 9-16-86; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-416]

Mississippi Power and Light Co. et al.; Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing; Correction

In the September 4, 1986 issue of the Federal Register, the date on page 31741, first column, second complete paragraph, reads October 7, 1986; it should be corrected to read October 6, 1986.

Dated at Bethesda, Maryland, this 12th day of September 1986.

For the Nuclear Regulatory Commission.

Donnie H. Grimsley,

Director, Division of Rules and Records, Office of Administration.

[FR Doc. 86-21064 Filed 9-16-86; 8:45 am]

BILLING CODE 7590-01-M

[Docket Nos. 50-352-OL, 50-353-OL]

Philadelphia Electric Co. (Limerick Generating Station, Units 1 and 2); Order (Time of Hearing Designated)

September 11, 1986.

Before Administrative Judges Helen F. Hoyt, Chairperson, Dr. Richard F. Cole, and Dr. Jerry Harbour.

The hearing on the remanded issue in ALAB-845, scheduled to be held on September 22, 1986 in the Conference Room, Independence Terrace—Room 7B, Holiday Inn, Midtown, 1305 Walnut Street, Philadelphia, Pennsylvania 19107, will be begin at 9:00 a.m.

For the Atomic Safety and Licensing Board.

Helen F. Hoyt,

Chairperson, Administrative Judge.

[FR Doc. 86-21065 Filed 9-16-86; 8:45 am]

BILLING CODE 7590-01-M

[Docket No. 50-26]

Carolina Power and Light Co.; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from the requirements of Appendix R to 10 CFR 50 to Carolina Power and Light Company (the licensee), for the H. B. Robinson Steam Electric Plant, Unit No. 2, located in Darlington County, South Carolina.

Environmental Assessment

Identification of Proposed Action

The exemption would:

1. Relieve the licensee from providing automatic fire suppression systems, pursuant to 10 CFR Part 50, Appendix R—section III.G.3 for all zones, and
2. Relieve the licensee from providing conventional radiant energy heat shields for cable protection, pursuant to 10 CFR Part 50, Appendix R—section III.G.2.f.

Equivalent levels of protection would be provided by the licensee.

The Need for the Proposed Action

The proposed exemption is needed in order to permit the licensee to use alternate fire protection configurations