September 26, 1990

Docket No. 50-423

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Mr. Edward J. Mroczka
Senior Vice President
Nuclear Engineering and Operations
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 77145)

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated July 10, 1990.

The amendment modifies Millstone Unit 3 Technical Specification (TS) Table 3.3-9, "Remote Shutdown Instrumentation," to correct an editorial error and TS Table 4.3-6, "Remote Shutdown Monitoring Instrumentation Surveillance Requirements," to provide a footnote concerning the source range count rate.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

David H. Jaffe, Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 56 to NPF-49
2. Safety Evaluation

cc w/enclosures:
See next page

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Millstone Nuclear Power Station
Unit No. 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated July 10, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 56 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ronald W. Hernan for

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 26, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILTIY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
3/4 3-54	3/4 3-54
3/4 3-58	3/4 3-58

TABLE 3.3-9

REMOTE SHUTDOWN INSTRUMENTATION

INSTRUMENT	READOUT LOCATION	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE
1. Reactor Trip Breaker Indication	Reactor Trip Switchgear	1/trip breaker	1/trip breaker
2. Pressurizer Pressure	Aux. Shutdown Panel	2	1
3. Pressurizer Level	Aux. Shutdown Panel	2	1
4. Steam Generator Pressure	Aux. Shutdown Panel	2/steam generator	1/steam generator
5. Steam Generator Water Level	Aux. Shutdown Panel	2/steam generator	1/steam generator
6. Auxiliary Feedwater Flow Rate	Aux. Shutdown Panel	1/steam generator	1/steam generator
7. Loop Hot Leg Temperature	Aux. Shutdown Panel	1/loop	1/loop
8. Loop Cold Leg Temperature	Aux. Shutdown Panel	1/loop	1/loop
9. Reactor Coolant System Pressure (Wide Range)	Aux. Shutdown Panel	2	1
10. DWST Level	Aux. Shutdown Panel	2	1
11. RWST Level	Aux. Shutdown Panel	2	1
12. Containment Pressure	Aux. Shutdown Panel	2	1
13. Emergency Bus Voltmeters	Aux. Shutdown Panel	1/train	1/train
14. Source Range Count Rate	Aux. Shutdown Panel	2	1
15. Intermediate Range Flux	Aux. Shutdown Panel	2	1
16. Boric Acid Tank Level	Aux. Shutdown Panel	2/tank	1/tank

TRANSFER SWITCHES	SWITCH LOCATION
1. Auxiliary Feedwater Isolation FWA*MOV35A	Transfer Switch Panel
2. Auxiliary Feedwater Isolation FWA*MOV35B	Transfer Switch Panel
3. Auxiliary Feedwater Isolation FWA*MOV35C	Transfer Switch Panel
4. Auxiliary Feedwater Isolation FWA*MOV35D	Transfer Switch Panel
5. Auxiliary Feedwater Pump Ah. Suction FWA*AOV23A	Transfer Switch Panel
6. Auxiliary Feedwater Pump Ah. Suction FWA*AOV23B	Transfer Switch Panel

TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Reactor Trip Breaker Indication	M	N.A.
2. Pressurizer Pressure	M	R
3. Pressurizer Level	M	R
4. Steam Generator Pressure	M	R
5. Steam Generator Water Level	M	R
6. Auxiliary Feedwater Flow Rate	M	R
7. Loop Hot Leg Temperature	M	R
8. Loop Cold Leg Temperature	M	R
9. Reactor Coolant System Pressure (Wide Range)	M	R
10. DWST Level	M	R
11. RWST Level	M	R
12. Containment Pressure	M	R
13. Emergency Bus Voltmeters	M	R
14. Source Range Count Rate	M*	R
15. Intermediate Range Amps	M	R
16. Boric Acid Tank Level	M	R

* When below P-6 (intermediate range neutron flux interlock setpoint).

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 56

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

INTRODUCTION

By application for license amendment dated July 10, 1990, Northeast Nuclear Energy Company, et al. (the licensee), requested changes to Millstone Unit 3 Technical Specifications (TS).

The proposed amendment would modify Millstone Unit 3 Technical Specification (TS) Table 3.3-9, "Remote Shutdown Instrumentation," to correct an editorial error and TS Table 4.3-6, "Remote Shutdown Monitoring Instrumentation Surveillance Requirements," to provide a note of clarification concerning the source range count rate.

DISCUSSION AND EVALUATION

An editorial error has been identified in TS Table 3.3-9 which resulted in the column headings for "Readout Location," "Total No. of Channels" and "Minimum Channels Operable" being misaligned. The proposed change to TS Table 3.3-9 would result in the column headings being correctly aligned. While the licensee has identified only two columns as being effected, all three columns would be changed.

The proposed change to Table 3.3-9 corrects the editorial error and does not otherwise change the requirements stated therein. Accordingly, the proposed change to TS Table 3.3-9 is acceptable.

The proposed change to TS Table 4.3-6 would add a footnote to the surveillance requirements for source range count rate channel check. At the present time TS Table 4.3-6 specifies a monthly interval for the source range count rate channel check. Since power to the source range detectors is removed in the intermediate neutron range (above the P-6 interlock) the source range detectors are effectively inoperable during power operation. Accordingly, a channel check of the source range count rate is meaningless during power operation. The licensee has proposed that the following footnote be added to the source range count rate channel check: When below P-6 (intermediate range neutron flux interlock setpoint).

The proposed footnote would have the effect of eliminating the requirement to perform a channel check on the source range count rate when the reactor is operating at power levels above the P-6 interlock.

The TS defines a "Channel Check" as follows:

> "1.6 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter."

Since power is removed from the source range channels above the P-6 interlock, a "Channel Check" of the source range channels would not provide a " ... qualitative assessment of channel behavior during operation by observation." Accordingly, the subject surveillance is meaningless and its elimination from the TS would not effect the availability of the source range monitors. Accordingly, the proposed change to the TS, which involves adding a footnote to TS Table 4.3-6 is acceptable.

ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 26, 1990

Principal Contributor:

D. Jaffe