

July 17, 2001

MEMORANDUM TO: Biweekly Notice Coordinator

FROM: Stephen P. Sands, Project Manager, Section 2 */RA/*  
Project Directorate III  
Division of Licensing Project Management

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE -  
NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT  
TO FACILITY OPERATING LICENSE, PROPOSED NO  
SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION,  
AND OPPORTUNITY FOR A HEARING (TAC NO. MB2107)

FirstEnergy Nuclear Operating Company, Docket No. 50-346, Davis-Besse Nuclear Power  
Station (DBNPS), Unit 1, Ottawa County, Ohio

Date of amendment request: May 22, 2001

Description of amendment request: The proposed amendment would revise the once-through steam generator (OTSG) tube repair roll requirements to (1) utilize updated limiting tensile tube loads, (2) define new exclusion zones within the steam generator in which the application of the repair roll is prohibited, (3) allow the repair roll to be used in the lower tubesheet area, (4) remove the limitation of only one repair roll per OTSG tube, and (5) replace the requirement that the repair roll be one inch in length with a requirement that the repair roll be installed in accordance with Framatome Technologies Incorporated Report BAW-2303P, Revision 4, "OTSG Repair Roll Qualification Report."

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensees have provided their analysis of the issue of no significant hazards consideration, which is presented below:

- 1a. Not involve a significant increase in the probability of an accident previously evaluated because testing and analysis have shown the once-through steam generator (OTSG) tube repair roll process under the proposed revised Technical specification (TS) Surveillance Requirement (SR) 4.4.5.4 ensures the new pressure boundary joint created by the repair roll process provides

adequate structural and leakage integrity for all normal operating and accident conditions. In addition, the removal of the name "Babcock & Wilcox" is an administrative change to reflect that Framatome ANP has succeeded the Babcock & Wilcox Company. Therefore, the proposed changes to SR 4.4.5.4 will not increase the probability of a previously evaluated accident.

The proposed change to TS Bases 3/4.4.5 reflects the changes proposed to its associated SR, and does not involve an increase in the probability of an accident previously evaluated.

- 1b. Not involve a significant increase in the consequences of an accident previously evaluated because the repair roll process under the proposed revised SR 4.4.5.4 ensures the new pressure boundary joint created by the repair roll process provides adequate structural and leakage integrity under all accident conditions. Any leakage resulting from repair roll joint slippage under accident conditions will be accounted for to ensure that the post-accident OTSG leakage will not exceed that assumed in the accident analyses. Should a repaired tube fail, the radiological consequences would be bounded by the existing Steam Generator Tube Rupture analysis.

The proposed change to Bases 3/4.4.5 reflects the changes proposed to its associated SR, and does not involve an increase to the consequences of an accident previously evaluated.

2. Not create the possibility of a new or different kind of accident from any accident previously evaluated because there will be no change in the operation of the steam generators or connecting systems as a result of the repair roll process added by the proposed changes to SR 4.4.5.4. The physical changes in the steam generators associated with the repair roll process have been evaluated and do not create the possibility for a new or different kind of accident from any accident previously evaluated, i.e., the physical change in the steam generators is limited to the location and accident slip behavior of the primary to secondary boundary within the tubesheet. Accordingly, these changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change to Bases 3/4.4.5 reflects the changes proposed to its associated SR, and does not create the possibility of any new or different kind of accident.

3. Not involve a significant reduction in a margin of safety because tubes with primary system to secondary system boundary joints created by the repair roll process have been shown by testing and analysis to satisfy all structural, leakage, and heat transfer requirements. The additional testing of tubes repaired by the repair roll process under existing SR 4.4.5.9 provides continuing inservice monitoring of these tubes such that inservice degradation of tubes repaired by the repair roll process will be detected.

Therefore, the changes to SR 4.4.5.4 to modify the repair process do not reduce and margin of safety.

The proposed change to Bases 3/4.4.5 reflects the changes proposed to its associated SR, and does not reduce the margin of safety.

On the basis of the above, the Davis-Besse Nuclear Power Station has determined that the License Amendment Request does not involve a significant hazards consideration. As this License Amendment Request concerns a proposed change to the Technical Specifications that must be reviewed by the Nuclear Regulatory Commission, this License Amendment Request does not constitute an unreviewed safety question.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Mary E. O'Reilly, Attorney, FirstEnergy Corporation, 76 South Main Street, Akron, OH 44308

NRC Section Chief: Anthony J. Mendiola

Therefore, the changes to SR 4.4.5.4 to modify the repair process do not reduce and margin of safety.

The proposed change to Bases 3/4.4.5 reflects the changes proposed to its associated SR, and does not reduce the margin of safety.

On the basis of the above, the Davis-Besse Nuclear Power Station has determined that the License Amendment Request does not involve a significant hazards consideration. As this License Amendment Request concerns a proposed change to the Technical Specifications that must be reviewed by the Nuclear Regulatory Commission, this License Amendment Request does not constitute an unreviewed safety question.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Mary E. O'Reilly, Attorney, FirstEnergy Corporation, 76 South Main Street, Akron, OH 44308

NRC Section Chief: Anthony J. Mendiola

Distribution:  
Non-Public  
PD3-2 r/f

ML011780178

OFFICE	PM:LPD3-2	LA:LPD3-2	SC:LPD3-2
NAME	SSands	THarris	AMendiola/ <i><b>JHopkins for/</b></i>
DATE	07/13/01	07/13/01	07/13/01

OFFICIAL RECORD COPY