Docket No. 50-423

Mr. Edward J. Mroczka
Senior Vice President
Nuclear Engineering and Operations
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: MILLSTONE UNIT 3 - ISSUANCE OF AMENDMENT (TAC NO. 74555)

The Commission has issued the enclosed Amendment No. 42 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3, in response to your application dated August 14 1989.

The amendment modifies Facility Operating License, NPF-49, by deleting the following license conditions: (1) 2.C(3), "Containment Average Temperature," (2) 2.C(4), "N-1 Loop Operation," (3) 2.C(6), "Instrumentation for Monitoring Post-Accident Conditions, R.G. 1.97, Revision 2 Requirements," (4) 2.C(8), "Moisture in Air Start System," (5) 2.C(g), "Operating Staff Experience Requirements," (6) 2.C(11), "Revised Small Break LOCA Methods to Show Compliance With 10 CFR 50.46, TMI Item II.K.3.31," (7) 2.C(12), "Safety Parameter Display System (SPDS)," and (8) 2.C(13), "Detailed Control Room Design Review."

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/ Signed by J.Stolz for

David H. Jaffe, Project Manager Project Directorate I-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Fnclosures:

1. Amendment No. 42 to NPF-49

2. Safety Evaluation

cc w/enclosures:
See next page

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DATED: December 18, 1989

AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. NPF-49

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cc: Licensee/Applicant Service List

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cc:

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Mr. Alan Menard, Manager Technical Services Massachusetts Municipal Wholesale Electric Company Post Office Box 426 Ludlow, Massachusetts 01056



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

#### DOCKET NO. 50-423

#### MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42 License No. NPF-49

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated August 14, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



# UNITED STATES NÚCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 42

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

#### INTRODUCTION

By application for license amendment dated August 14, 1989, Northeast Nuclear Energy Company, et al. (the licensee), requested changes to Millstone Unit 3 Facility Operating License NPF-49. The proposed amendment would modify the Facility Operating License, NPF-49, by deleting the following license conditions: (1) 2.C(3), "Containment Average Temperature," (2) 2.C(4), "N-1 Loop Operation," (3) 2.C(6), "Instrumentation for Monitoring Post-Accident Conditions, R.G. 1.97, Revision 2 Requirements," (4) 2.C(8), "Moisture in Air Start System," (5) 2.C(9), "Operating Staff Experience Requirements," (6) 2.C(11), "Revised Small Break LOCA Methods to Show Compliance With 10 CFR 50.46, TMI Item II.K.3.31," (7) 2.C(12), "Safety Parameter Display System (SPDS)," and (8) 2.C(13), "Detailed Control Room Design Review."

The licensee has proposed that the above license conditions be deleted in that their subject provisions have been satisfied.

#### DISCUSSION AND EVALUATION

On January 31, 1986, the NRC staff issued Facility Operating License No. NPF-49 to Northeast Nuclear Energy Company, et. al. (the licensee) for operation of Millstone Unit No. 3. As part of NPF-49, the NRC specified that certain actions, "License Conditions," were to be undertaken by the licensee. The license condition numbers correspond to sections within NPF-49. The satisfaction of these conditions is described below.

### 1. <u>License Condition 2.C(3)--Containment Average Temperature</u>

License Condition 2.C(3) in the Millstone Unit No. 3 Operating License requires that "Prior to start-up following the first refueling outage, the qualified life of electrical equipment within the scope of 10 CFR 50.49 located inside containment shall be recalculated based on the actual temperature monitored inside containment during the first cycle of operation." In a letter dated October 3, 1988, the NRC concluded that License Condition 2.C(3) had been met.

#### 2. 2.C(4)--N-1 Loop Operation

License Condition 2.C(4) in the Millstone Unit No. 3 operating license prohibits three-loop operations until outstanding issues are resolved to the satisfaction of the NRC Staff. The NRC reviewed information presented by the licensee and concluded in a letter dated November 16, 1987 that the Staff's concerns were satisfied and the license condition was met.

# 3. 2.C(6)--Instrumentation for Monitoring Post-Accident Conditions R.G 1.97, Revision 2 Requirements

License Condition 2.C(6) requires that "NNECO shall install and have operational instrumentation to monitor containment sump water temperature at the first scheduled outage of sufficient duration after procurement is complete, but no later than start-up after the first refueling outage." In a letter dated October 21, 1987, the NRC informed NNECO that the Staff is currently generically reviewing the need for Category 2 instrumentation to monitor the containment sump water temperature, and the use of Category 3 instrumentation was acceptable for operation while their review is ongoing. In addition, the Staff concluded that License Condition 2.C(6) was satisfied.

#### 4. 2.C(8)--Mositure in Air Start System

License Condition 2.C(8) requires that "NNECO shall install air dryers in the emergency diesel air start system at the first scheduled outage of sufficient duration after procurement is completed, but no later than start-up after the first refueling outage." In response to this license condition, air drying skids were installed in each emergency diesel generator starting air system and made operational during the first refueling outage. In a letter dated January 31, 1988, NNECO confirmed the completion of actions to satisfy License Condition 2.C(8). Subsequently, on February 8, 1988, the NRC issued Routine Inspection (RI) Report 50-423/87-30. Section 4.2 of RI 50-423/87-30 describes the NRC staff inspection of the emergency diesel generator starting air dryers and concludes that they, "...fulfilled the conditions of Operating License condition 2.C(8)."

### 5. 2.C(9)--Operating Staff Experience Requirements

License Condition 2.C(9) specifies that each operating shift shall have a licensed senior operator or an advisor who had certain experience in the operation of facilities similar to Millstone Unit No. 3. In a letter dated July 9, 1986, the NRC confirmed that NNECO met license condition 2.C(9).

# 6. 2.C(11)--Revised Small Break LOCA Methods to Show Compliance With 10 CFR 50.45, TMI Item II.K.3.31

License Condition 2.C(11) required plant-specific analysis using NOTRUMP or a generic submittal referencing NOTRUMP. In a letter dated January 13, 1987, the NRC indicated that the Staff has reviewed WCAP 22245 and the information submitted in the licensee's letter dated June 6, 1986, and concluded that License Condition 2.C(11) had been met.

#### 7. 2.C(12)--SPDS

License Condition 2.C(12) required that: "Prior to restart following the first refueling outage, NNECO shall add to the Safety Parameter Display System (SPDS) and have operational the following SPDS parameters:

(i) Residual Heat Removal (RHR) flow

(ii) Containment Isolation

(iii) Containment Hydrogen Concentration

(iv) Primary Coolant System Hot Leg Temperature"

In a letter dated March 7, 1988, the NRC staff concluded that License Condition 2.C(12) had been satisfied.

## 8. 2.C(13)--Detailed Control Room Design Review

License Conditon 2.C(13) required that "NNECO shall correct all human engineering discrepancies (HEDs) not resolved prior to exceeding 5 percent power, and according to the schedule specified in a letter from J. F. Opeka to V. S. Noonan dated January 29, 1986." In a letter dated January 13, 1987, the NRC concluded that License Condition 2.C(13) of the Millstone Unit No. 3 operating license had been satisfied.

Since all of the above described license conditions have been fulfilled by the licensee, and the NRC staff agrees that they have been fulfilled, deletion of these conditions from the license will have no effect on the safe operation of Millstone Unit 3 and is acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: Amendment No. 42

Principal Contributor: D. H. Jaffe

- 2. Accordingly, Facility Operating License No. NPF-49 is hereby amended by deletion of Sections 2.C(3), 2.C(4), 2.C(6), 2.C(8), 2.C(9), 2.C(11), 2.C(12) and 2.C(13).\*
- 3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

You F. Stolz, Director Project Directorate I=4

Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Pages 4 and 6 of license

Date of Issuance: December 18, 1989

<sup>\*</sup>Pages 4 and 6 are attached, for conveinience, for the composite license to reflect this change.

- (3) deleted
- (4) deleted
- (5) <u>Inservice Inspected Program Section 5.2.4.3, SER, 6.6.3 SER)</u>

Prior to May 25, 1986, NNECO shall submit the inservice inspection program which conforms to the ASME Code in effect on November 25, 1984 in accordance with 50.55a(g)(4), for NRC staff review and approval.

(6) deleted

## - 6 - (next page is 8)

- (8) deleted
- (9) deleted
- (10) deleted
- (11) deleted
- (12) deleted
- (13) deleted
- (14) Salem ATWS Events Generic Letter 83-28

NNECO shall submit responses to and implement the requirements of Generic Letter 83-28 Salem ATWS Events on a schedule which is consistent with that given in their November 8, 1983, letter as modified by their letters dated March 16, 1984 and September 5, 1985.