



Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000

June 15, 2001

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

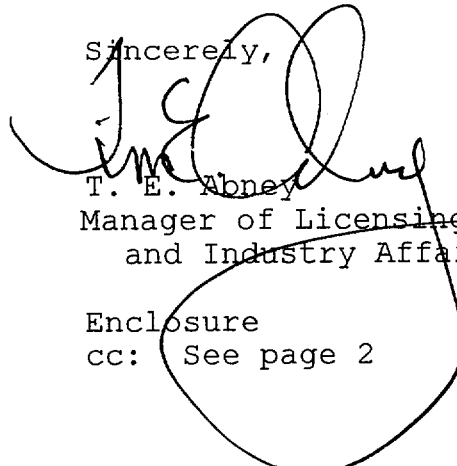
In the Matter of)	Docket Nos.	50-259
Tennessee Valley Authority)		50-260
			50-296

BROWNS FERRY NUCLEAR PLANT (BFN) - MAY 2001 MONTHLY OPERATING REPORT

The Enclosure provides the May 2001 Monthly Operating Report (MOR) as required by BFN Technical Specifications Section 5.6.4.

In accordance with NRC RIS 2001-05, only one paper copy of this document is being sent to the NRC Document Control Desk. If you have any questions concerning this report, please call me at (256) 729-2636.

Sincerely,



T. E. Abney
Manager of Licensing
and Industry Affairs

Enclosure
cc: See page 2

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U.S. Nuclear Regulatory Commission
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June 15, 2001

Enclosure

cc (Enclosure):

INPO Records Center
Institute of Nuclear Power Operations
700 Galleria Parkway
Atlanta, Georgia 30339-5957

Mr. James Lang, Manager
Advanced Reactor Department
Electric Power Research Institute
3340 Hillview Avenue
Palo Alto, California 94304

Ms. Barbara Lewis
McGraw-Hill Companies
1200 G Street, N.W.
Suite 1100
Washington, D.C. 20005-3802

(Via NRC Electronic Distribution)

Mr. Paul E. Fredrickson, Branch Chief
U.S. Nuclear Regulatory Commission
Region II
Sam Nunn Atlanta Federal Center
61 Forsyth Street, S.W., Suite 23T85
Atlanta, Georgia 30303

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Project Directorate II
Division of Licensing Project Management
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NRC Resident Inspector
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10833 Shaw Road
Athens, Alabama 35611

Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
Sam Nunn Atlanta Federal Center
61 Forsyth Street, S.W., Suite 23T85
Atlanta, Georgia 30303

ENCLOSURE 1

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)

MONTHLY OPERATING REPORT

MAY 2001

UNIT 1

DOCKET NUMBER 50-259

LICENSE NUMBER DPR-33

UNIT 2

DOCKET NUMBER 50-260

LICENSE NUMBER DPR-52

UNIT 3

DOCKET NUMBER 50-296

LICENSE NUMBER DPR-68

OPERATIONAL SUMMARY
MAY 2001

BROWNS FERRY NUCLEAR PLANT UNIT 1

Unit 1 remains shutdown on administrative hold to resolve various TVA and NRC concerns. Unit 1 has been on administrative hold since June 1, 1985. As a result, TVA considers that accrual of reporting hours is suspended since the unit has a maximum dependable capacity (MDC) of zero MWe. Accordingly, TVA does not report cumulative hours for the period beginning June 1, 1985, when calculating the operating status variables.

BROWNS FERRY NUCLEAR PLANT UNIT 2

For the month of May, Unit 2 operated at approximately 100 percent power.

BROWNS FERRY NUCLEAR PLANT UNIT 3

For the month of May, Unit 3 operated at approximately 100 percent power.

OPERATING DATA REPORT

Docket No. 50-259
 Unit Name BFN Unit 1
 Date June 4, 2001
 Completed By J. E. Wallace
 Telephone (256) 729-7874
 Reporting Period May 2001

1. Design Electrical Rating (Net MWe): 1065
2. Maximum Dependable Capacity (MWe-Net) 0

	Month	Yr-to-Date	Cumulative*
3. Number of Hours Reactor was Critical	0	0	59521
4. Hours Generator On-Line	0	0	58267
5. Unit Reserve Shutdown Hours	0	0	0
6. Net Electrical Energy Generated (MWh)	0	0	53,796,427

* Excludes hours under Administration Hold (June 1, 1985 to Present)

OPERATING DATA REPORT

Docket No. 50-260
 Unit Name BFN Unit 2
 Date June 4, 2001
 Completed By J. E. Wallace
 Telephone (256) 729-7874
 Reporting Period May 2001

1. Design Electrical Rating (Net MWe): 1120
2. Maximum Dependable Capacity (MWe-Net) 1118

	Month	Yr-to-Date	Cumulative
3. Number of Hours Reactor was Critical	744	2,779	135,088
4. Hours Generator On-Line	744	2,639	132,495
5. Unit Reserve Shutdown Hours	0	0	0
6. Net Electrical Energy Generated (MWh)	835,044	2,859,839	130,686,745

OPERATING DATA REPORT

Docket No. 50-296
Unit Name BFN Unit 3
Date June 4, 2001
Completed By J. E. Wallace
Telephone (256) 729-7874
Reporting Period May 2001

1. Design Electrical Rating (Net MWe): 1120
2. Maximum Dependable Capacity (MWe-Net) 1118

	Month	Yr-to-Date	Cumulative
3. Number of Hours Reactor was Critical	744	3,623	91,693
4. Hours Generator On-Line	744	3,623	90,279
5. Unit Reserve Shutdown Hours	0	0	0
6. Net Electrical Energy Generated (MWh)	828,573	4,071,580	90,991,015

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT MONTH: MAY 2001

DOCKET NO: 50-259
UNIT NAME: BFN-1
DATE: June 4, 2001
COMPLETED BY: J. E. Wallace
TELEPHONE: (256) 729-7874

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁴	Cause and Corrective Action to Prevent Recurrence
1	06/01/85	S	744	F	4	N/A	N/A	N/A	Administrative hold to resolve various TVA and NRC concerns.

¹ **F: Forced**
S: Scheduled

² **Reason:**
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License
Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ **Method**
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing
Outage
5-Reduction
9-Other

⁴ **Instructions for Preparation of
Data Entry sheets for Licensee
Event Report (LER)**
(NUREG - 1022)

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT MONTH: MAY 2001

DOCKET NO: 50-260
UNIT NAME: BFN-2
DATE: June 4, 2001
COMPLETED BY: J. E. Wallace
TELEPHONE: (256) 729-7874

No.	Date	Type	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁴	Cause and Corrective Action to Prevent Recurrence
N/A									

¹ **F: Forced**
S: Scheduled

² **Reason:**
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License
Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ **Method**
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing
Outage
5-Reduction
9-Other

⁴ **Instructions for Preparation of
Data Entry sheets for Licensee
Event Report (LER)**
(NUREG - 1022)

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT MONTH: MAY 2001

DOCKET NO: 50-296
UNIT NAME: BFN-3
DATE: June 4, 2001
COMPLETED BY: J. E. Wallace
TELEPHONE: (256) 729-7874

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁴	Cause and Corrective Action to Prevent Recurrence
N/A									

¹ **F: Forced**
S: Scheduled

² **Reason:**
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License
Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ **Method**
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing
Outage
5-Reduction
9-Other

⁴ **Instructions for Preparation of
Data Entry sheets for Licensee
Event Report (LER)**
(NUREG - 1022)

June 25, 2001

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Subject: **Docket Nos. 50-361 and 50-362**
Proposed Change Number NPF-10/15-514
Increase in Reactor Power to 3438 MWt
San Onofre Nuclear Generating Station, Units 2 and 3

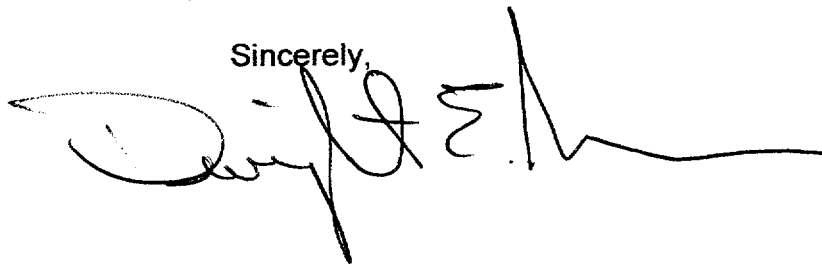
Reference: SCE to NRC letter dated April 3, 2001, Subject: Proposed Change
Number NPF-10/15-514 Increase in Reactor Power to 3438 MWt, San
Onofre Nuclear Generating Station Units 2 and 3

Gentlemen:

This letter provides responses to NRC requests for additional information (RAIs) concerning the Southern California Edison (SCE) request to increase the reactor power to 3438 MWt at San Onofre Units 2 and 3, Amendment Applications 207 and 192, Proposed Change Number 514 (Reference).

If you have any further questions regarding these amendment applications, please contact me or Mr. Jack L. Rainsberry (949) 368-7420.

Sincerely,



Enclosure

cc: E. W. Merschoff, Regional Administrator, NRC Region IV
C. C. Osterholtz, NRC Senior Resident Inspector, San Onofre Units 2 and 3
J. E. Donoghue, NRC Project Manager, San Onofre Units 2 and 3
S. Y. Hsu, Department of Health Services, Radiologic Health Branch


UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

Application of SOUTHERN CALIFORNIA)	
EDISON COMPANY, <u>ET AL.</u> for a Class 103)	Docket No. 50-361
License to Acquire, Possess, and Use)	
a Utilization Facility as Part of)	Amendment Application
Unit No. 2 of the San Onofre Nuclear)	No. 207
Generating Station)	

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL. pursuant to 10 CFR 50.90,
hereby submit information in support of Amendment Application No. 207. This
information consists of responses to NRC requests for additional information on
Proposed Change No. NPF-10-514 to Facility Operating License NPF-10. Proposed
Change No. NPF-10-514 is a request to revise the Facility Operating License by
increasing the licensed power for operation.

Subscribed on this 25th day of June, 2001.

Respectfully submitted,
SOUTHERN CALIFORNIA EDISON COMPANY

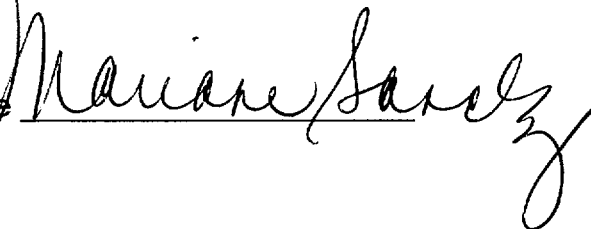
By: 
Dwight E. Nunn
Vice President

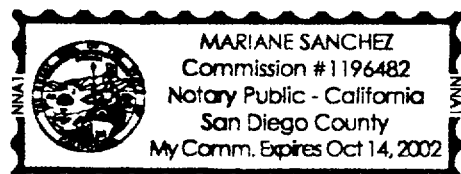
State of California

County of San Diego

On 6/25/2001 before me, Mariane Sanchez, personally
appeared Dwight E. Nunn, personally known to me to be the person whose
name is subscribed to the within instrument and acknowledged to me that he executed
the same in his authorized capacity, and that by his signature on the instrument the
person, or the entity upon behalf of which the person acted, executed the instrument.
WITNESS my hand and official seal.

Signature





UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

Application of SOUTHERN CALIFORNIA)	
EDISON COMPANY, <u>ET AL.</u> for a Class 103)	Docket No. 50-362
License to Acquire, Possess, and Use)	
a Utilization Facility as Part of)	Amendment Application
Unit No. 3 of the San Onofre Nuclear)	No. 192
Generating Station)	

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL. pursuant to 10 CFR 50.90,
hereby submit information in support of Amendment Application No. 192. This
information consists of responses to NRC requests for additional information on
Proposed Change No. NPF-15-514 to Facility Operating License NPF-15. Proposed
Change No. NPF-15-514 is a request to revise the Facility Operating License by
increasing the licensed power for operation.

Subscribed on this 25th day of June, 2001.

Respectfully submitted,
SOUTHERN CALIFORNIA EDISON COMPANY

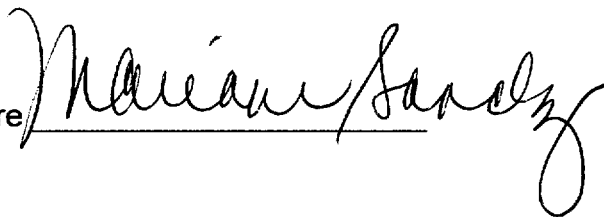
By: 
Dwight E. Nunn
Vice President

State of California

County of San Diego

On 6/25/2001 before me, Mariane Sanchez personally
appeared Dwight E. Nunn personally known to me to be the person whose
name is subscribed to the within instrument and acknowledged to me that he executed
the same in his authorized capacity, and that by his signature on the instrument the
person, or the entity upon behalf of which the person acted, executed the instrument.
WITNESS my hand and official seal.

Signature





Enclosure

Question 1:

Provide details of initial manual operation of power uprate.

Response:

The initial increase in reactor power, using the Crossflow system on San Onofre Units 2 and 3, is planned to be controlled manually by adjusting the correction factors, rather than by using the automatic update process discussed in Proposed Change Number (PCN)-514 and subsequent Southern California Edison responses to NRC requests for additional information. The manual adjustment is a calibration. The correction factors will be determined in the same manner as the automatic update, using the ratio of the process feedwater flow, steam flow, blowdown flow, feedwater temperature, and the Crossflow measurements of the parameters. The process data and Crossflow data will be taken at the same time and over several hours. The correction factors will then be manually input to the Plant Monitoring System (PMS) and Core Operating Limit Supervisory System (COLSS) Backup Computer. The measurements, calculation of correction factors, and manual input will be controlled by procedures. Persons performing and checking the measurements will initial or sign steps in the procedure.

At least one calibration will occur within a day prior to and following the initial increase in power. A second calibration will occur approximately two days later, then a third calibration approximately one week later. Subsequent calibrations will be performed at approximately every two weeks while in the manual mode. For San Onofre Unit 3, it is anticipated that the system will be available for service in the manual mode in early July 2001 and can be placed in automatic operation approximately the first week in September. For San Onofre Unit 2, it is anticipated that the system will be available for service in the manual mode in mid-August, 2001, and operated in automatic by approximately mid-September.

During the initial manual mode, the COLSS calculations will be monitored approximately daily to ensure that the process feedwater and steam flow venturis and transmitters are not drifting unexpectedly or excessively. Monitoring will include comparing the calculation of feedwater and steam flow by COLSS with the Crossflow system and comparing the calculations of reactor power by feedwater flow and main steam flow for divergences.

Question 2:

Confirm that the dose calculations and assessments supporting PCN-514 were all done using the analyses of record (AOR) dose calculation methodologies and that during the dose impact analyses/assessments, no dose analysis parameters were changed.

Response:

The dose calculations and assessments supporting Proposed Change Number (PCN)-514 were all done using the analyses of record dose calculation methodologies and dose analysis parameters, to the extent of the exceptions listed in Section 4.2 of the Amendment request. The exceptions are identified below.

Section	Event	Exceptions	Dose Impact
4.2.1.1	Inadvertent Opening of a Steam Generator Atmospheric Dump Valve with a Single Active Failure (IOSGADV/SAF)	None	N/A
4.2.1.2	Increased Main Steam Flow with a Single Active Failure (IMSF/SAF)	Power Uprate Source Term	AOR doses remain bounding
4.2.2.1	Limiting Fault events without fuel failure	None	N/A
4.2.2.2	Pre-trip Steam Line Break (SLB)	Power Uprate Source Term	AOR doses remain bounding
4.2.2.3	Reactor Coolant Pump (RCP) Sheared Shaft (SS)	Power Uprate Source Term	AOR doses remain bounding
4.2.2.4	Control Element Assembly Ejection (CEA Ej)	Power Uprate Source Term	Power Uprate thyroid doses are approximately 1% more severe. (Use of approved International Commission on Radiological Protection (ICRP)-30 Dose Conversion Factor (DCF) methodology would compensate by lowering thyroid doses by approximately 30%.)

Section	Event	Exceptions	Dose Impact
4.2.2.5	Loss Of Coolant Accident (LOCA)	Power Uprate Source Term	Power Uprate doses are more severe. (Per Table 4-1 of PCN-514 Amendments request)
4.2.2.6	Fuel Handling Accident (FHA)	Power Uprate Source Term	Power Uprate doses are 2% more severe. (Removal of excess modeling conservatism present in the pairings of radial peaking factors and iodine fuel rod gap release fractions would compensate by lowering the doses.)
4.2.2.7	Spent Fuel Pool (SFP) Boiling	Power Uprate Source Term; SFP Heat Load; Initial SFP water temperature	AOR doses remain bounding
4.2.3.1	Equipment Qualification (EQ) Doses	Power Uprate Source Term	AOR doses remain bounding
4.2.3.2	Radwaste	Power Uprate Source Term	AOR doses remain bounding

Question 3:

Is there any change in operator action times as a result of PCN-514 power uprate?

Response:

The timing of operator actions supporting Updated Final Safety Analysis Report (UFSAR) Chapter 15 accident analyses are not changed as a result of PCN-514 power uprate.