

May 13, 1987

Doc 016

Docket No. 50-423

DISTRIBUTION:

Docket File
NRC & LPDRs
ARM/LFMB
PD I-4 Files
RFerguson
WJones
EJordan
DHagan

SVarga
GPA/PA
EButcher
BBoger
SNorris
JPartlow
TBarnhart-4
OGC-Beth
ACRS 10

Mr. Edward J. Mroczka
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, CT 06141-0270

Dear Mr. Mroczka:

SUBJECT: AMENDMENT NO. 4 TO FACILITY OPERATING LICENSE NO. NPF-49
(TAC# 64563)

The Commission has issued the enclosed Amendment No. 4 to Facility Operating License No. NPF-49 for Millstone Nuclear Power Station, Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application dated January 30, 1987, and supplemental letter dated March 6, 1987.

This amendment would extend, on a one-time basis, the diesel generator 18-month inspection required in Technical Specification 4.8.1.1.2.f.1) and 4.0.6 from May 1987 to the refueling outage scheduled to start about January 1988.

A copy of the related Safety Evaluation also is enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

Robert Ferguson, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 4 to NPF-49.
2. Safety Evaluation

cc w/enclosures:
See next page

8705190144 870513
PDR ADDCK 05000423
P PDR

PDI-4
SNorris
5/5/87

PDI-4
RFerguson;eh
5/5/87

PDI-4
JS to lz
5/5/87

OGC
M. Korman
5/16/87

Mr. E. J. Mroczka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit No. 3

cc:
Gerald Garfield, Esq.
Day, Berry & Howard
City Place
Hartford, Connecticut 06103-3499

Ms. Jane Spector
Federal Energy Regulatory Commission
825 N. Capitol Street, NE
Room 8608C
Washington, D.C. 20426

Mr. Maurice R. Scully, Executive
Director
Connecticut Municipal Electric
Energy Cooperative
268 Thomas Road
Groton, Connecticut 06340

Mr. Kevin McCarthy, Director
Radiation Control Unit
Department of Environmental Protection
State Office Building
Hartford, Connecticut 06115

Robert W. Bishop, Esq.
Corporate Secretary
Northeast Utilities
Post Office Box 270
Hartford, Connecticut 06141

Burlington Electric Department
c/o Robert F. Fletcher, Esq.
271 South Union Street
Burlington, Vermont 05402

Mr. Thomas Shedlosky
Senior Resident Inspector Office
U. S. Nuclear Regulatory Commission
Millstone III
P. O. Box 811
Niantic, Connecticut 06357-08

Mr. Michael L. Jones, Manager
Project Management Department
Massachusetts Municipal Wholesale
Electric Company
Post Office Box 426
Ludlow, Massachusetts 01056

Regional Administrator
U. S. NRC, Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Mr. Karl Abraham
Public Affairs Office, Region I
U. S. Nuclear Regulatory Commission,
King of Prussia, Pennsylvania 19406



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.*

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al., (the licensee) dated January 30, 1987, as supplemented March 6, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*Northeast Nuclear Energy Company is authorized to act as agent and representative for the following Owners: Central Maine Power Company, Central Vermont Public Service Corporation, Chicopee Municipal Lighting Plant, City of Burlington, Vermont, Connecticut Municipal Electric Light Company, Massachusetts Municipal Wholesale Electric Company, Montaup Electric Company, New England Power Company, The Village of Lyndonville Electric Department, Western Massachusetts Electric Company, and Vermont Electric Generation and Transmission Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

8705190151 870513
PDR ADOCK 05000423
P PDR

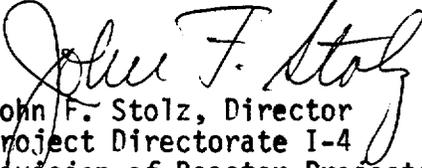
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 4 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 4

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3/4 0-3
3/4 8-4

Insert Pages

3/4 0-3
3/4 8-4

APPLICABILITY

LIMITING CONDITION FOR OPERATION (Continued)

- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements; and
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

4.0.6 By specific reference to this section, those surveillances which must be performed on or before May 1987, and are designated as 18-month surveillances (or required as outage-related surveillance under the provisions of Specification 4.0.5) may be delayed until the beginning of the refueling outage scheduled to begin about January 1988. For these specific surveillances under this section, the specified time intervals required by Specification 4.0.2 will be determined with the new initiation date established by the surveillance date during the Millstone Unit No. 3 refueling outage.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS by:*
- 1) Verifying the fuel level in the day tank,
 - 2) Verifying the fuel level in the fuel storage tank,
 - 3) Verifying the fuel transfer pump starts and transfers fuel from the storage system to the day tank,
 - 4) Verifying the lubricating oil inventory in storage,
 - 5) Verifying the diesel starts from ambient condition and accelerates to at least 508 rpm in less than or equal to 10 seconds. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 0.8 Hz within 10 seconds after the start signal. The diesel generator shall be started for this test by using one of the following signals:
 - a) Manual, or
 - b) Simulated loss-of-offsite power by itself, or
 - c) Simulated loss-of-offsite power in conjunction with an ESF Actuation test signal, or
 - d) An ESF Actuation test signal by itself.
 - 6) Verifying the generator is synchronized, loaded to greater than or equal to 4986 kW in less than or equal to 60 seconds, and operates with a load greater than or equal to 4986 kW for at least 60 minutes, and
 - 7) Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 31 days and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the day tank;
- c. At least once per 31 days by checking for and removing accumulated water from the fuel oil storage tanks;
- d. By sampling new fuel oil in accordance with ASTM-D4057 prior to addition to storage tanks and:
- 1) By verifying in accordance with the tests specified in ASTM-D975-81 prior to addition to the storage tanks that the sample has:

*All diesel generator starts for the purpose of this surveillance test may be preceded by an engine prelube period. Further, all surveillance tests, with the exception of once per 184 days, may also be preceded by warmup procedures (e.g., gradual acceleration and/or gradual loading >60 sec) as recommended by the manufacturer so that the mechanical stress and wear on the diesel engine is minimized.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- a) An API Gravity of within 0.3 degrees at 60°F, or a specific gravity of within 0.0016 at 60/60°F, when compared to the supplier's certificate, or an absolute specific gravity at 60/60°F of greater than or equal to 0.83 but less than or equal to 0.89, or an API gravity of greater than or equal to 27 degrees but less than or equal to 39 degrees;
 - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes (alternatively, Saybolt viscosity, SUS at 100°F of greater than or equal to 32.6, but not less than or equal to 40.1), if gravity was not determined by comparison with the supplier's certification;
 - c) A flash point equal to or greater than 125°F; and
 - d) A clear and bright appearance with proper color when tested in accordance with ASTM-D4176-82.
- 2) By verifying within 30 days of obtaining the sample that the other properties specified in Table 1 of ASTM-D975-81 are met when tested in accordance with ASTM-D975-81 except that the analysis for sulfur may be performed in accordance with ASTM-D1552-79 or ASTM-D2622-82.
- e. At least once every 31 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-78, and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM-D2276-78, Method A;
 - f. At least once per 18 months, during shutdown, by:
 - 1) Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service;*
 - 2) Verifying the generator capability to reject a load of greater than or equal to 595 kW while maintaining voltage at 4160 ± 420 volts and frequency at 60 ± 3 Hz;
 - 3) Verifying the generator capability to reject a load of 4986 kW without tripping. The generator voltage shall not exceed 4784 volts during and following the load rejection;
 - 4) Simulating a loss-of-offsite power by itself, and:
 - a) Verifying deenergization of the emergency busses and load shedding from the emergency busses, and

*The provisions of Specification 4.0.6 are applicable.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- a) An API Gravity of within 0.3 degrees at 60°F, or a specific gravity of within 0.0016 at 60/60°F, when compared to the supplier's certificate, or an absolute specific gravity at 60/60°F of greater than or equal to 0.83 but less than or equal to 0.89, or an API gravity of greater than or equal to 27 degrees but less than or equal to 39 degrees;
 - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes (alternatively, Saybolt viscosity, SUS at 100°F of greater than or equal to 32.6, but not less than or equal to 40.1), if gravity was not determined by comparison with the supplier's certification;
 - c) A flash point equal to or greater than 125°F; and
 - d) A clear and bright appearance with proper color when tested in accordance with ASTM-D4176-82.
- 2) By verifying within 30 days of obtaining the sample that the other properties specified in Table 1 of ASTM-D975-81 are met when tested in accordance with ASTM-D975-81 except that the analysis for sulfur may be performed in accordance with ASTM-D1552-79 or ASTM-D2622-82.
- e. At least once every 31 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-78, and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM-D2276-78, Method A;
 - f. At least once per 18 months, during shutdown, by:
 - 1) Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service;*
 - 2) Verifying the generator capability to reject a load of greater than or equal to 595 kW while maintaining voltage at 4160 ± 420 volts and frequency at 60 ± 3 Hz;
 - 3) Verifying the generator capability to reject a load of 4986 kW without tripping. The generator voltage shall not exceed 4784 volts during and following the load rejection;
 - 4) Simulating a loss-of-offsite power by itself, and:
 - a) Verifying deenergization of the emergency busses and load shedding from the emergency busses, and

*The provisions of Specification 4.0.6 are applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 4

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

By letter dated January 30, and supplemented by letter dated March 6, 1987, the Northeast Nuclear Energy Company (licensee) requested a change to the Millstone Unit 3 Technical Specifications, Section 4.0.6 and Surveillance Requirement 4.8.1.1.2.f.1). The proposed change would extend, on a one-time basis, the diesel generator (DG) 18-month surveillance inspection up to 4 months beyond the maximum interval allowed by Technical Specification 4.0.6. The licensee states that, because of the extensiveness of this type of inspection, it should be performed during a refueling outage. Because the 18-month surveillance for the DG inspection occurs in May 1987, and the refueling outage is not scheduled until January 1988, the licensee cannot postpone the subject inspection without violating Technical Specification 4.0.2. Subsequent 18-month DG inspections would coincide with scheduled refueling outages.

2.0 EVALUATION

The subject inspection involves a partial teardown of the DG, which would render the DG inoperable longer than the 72-hour period prescribed by the Technical Specification, Limiting Conditions for Operation (LCO). The March 6, 1987 letter states that the subject inspection would require the DG to be in an inoperable state for approximately 12 days. Based on the extent of the DG inspection, the staff agrees that 12 days is a reasonable estimate for completing the work. Additionally, the licensee states that the DGs, which are used for standby service, log very few hours of operation on a yearly basis; thus engine wear and tear is minimal. The DG manufacturer concurs with the licensee's conclusion.

Because the inspection necessitates an extended shutdown of the reactor, the staff concurs with the licensee that the 18-month DG inspection should be performed during the Millstone Unit 3 refueling outage. Furthermore, the staff also concurs with the licensee that an 8-month extension of the surveillance interval will not result in excessive wear and tear on the diesel. Therefore, the staff finds the Technical Specification change requested in the January 30, 1987 letter, acceptable.

8705190153 870513
PDR ADOCK 05000423
P PDR

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 13, 1987

Principal Contributors:

R. Ferguson
R. J. Giardina