

August 22, 1995

Mr. Leon R. Eliason  
Chief Nuclear Officer & President-  
Nuclear Business Unit  
Public Service Electric & Gas  
Company  
Post Office Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION (TAC NO. M91573)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment No. 77 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 13, 1995.

This amendment revises the Administrative Controls Section (6.0) of the Technical Specifications (TS) for Hope Creek Generating Station to reflect organizational changes and resultant management title changes.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

David H. Jaffe, Senior Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Amendment No. 77 to  
License No. NPF-57  
2. Safety Evaluation

cc w/encs: See next page

DISTRIBUTION

Docket File	JStolz	GHill(2)	OC/LFDCB
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JZwolinski	OPA		

\*See previous concurrence

OFFICE	PDI-2/AA	PDI-2/PM	PDI-2/PM	*OGC	PDI-2/D
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DATE	8/21/95	8/21/95	8/21/95	06/20/95	8/21/95

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DOCUMENT NAME: HC91573.AMD



UNITED STATES  
**NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

August 22, 1995

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Public Service Electric & Gas  
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Sincerely,

A handwritten signature in black ink, appearing to read "David H. Jaffe", is written over a circular stamp or seal.

David H. Jaffe, Senior Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Amendment No. 77 to  
License No. NPF-57  
2. Safety Evaluation

cc w/encs: See next page

Mr. Leon R. Eliason  
Public Service Electric & Gas  
Company

Hope Creek Generating Station

cc:

M. J. Wetterhahn, Esquire  
Winston & Strawn  
1400 L Street, N.W.  
Washington, DC 20005-3502

Ms. P. J. Curham  
MGR. Joint Generation Department  
Atlantic Electric Company  
Post Office Box 1500  
6801 Black Horse Pike  
Pleasantville, New Jersey 08232

R. Fryling, Jr., Esquire  
Law Department - Tower 5E  
80 Park Place  
Newark, New Jersey 07101

Richard Hartung  
Electric Service Evaluation  
Board of Regulatory Commissioners  
2 Gateway Center, Tenth Floor  
Newark, NJ 07102

Hope Creek Resident Inspector  
U.S. Nuclear Regulatory Commission  
Drawer I  
Hancocks Bridge, New Jersey 08038

Lower Alloways Creek Township  
c/o Mary O. Henderson, Clerk  
Municipal Building, P.O. Box 157  
Hancocks Bridge, NJ 08038

Mr. J. Hagan  
Vice President - Nuclear Operations  
Nuclear Department  
P.O. Box 236  
Hancocks Bridge, New Jersey 08038

Mr. S. LaBruna  
Vice President - Nuclear Engineering  
Nuclear Department  
P.O. Box 236  
Hancocks Bridge, New Jersey 08038

Mr. R. Hovey  
General Manager - Hope Creek Operations  
Hope Creek Generating Station  
P.O. Box 236  
Hancocks Bridge, New Jersey 08038

Mr. Frank X. Thomson, Jr., Manager  
Licensing and Regulation  
Nuclear Department  
P.O. Box 236  
Hancocks Bridge, New Jersey 08038

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Dr. Jill Lipoti, Asst. Director  
Radiation Protection Programs  
NJ Department of Environmental  
Protection and Energy  
CN 415  
Trenton, New Jersey 08625-0415



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. NPF-57

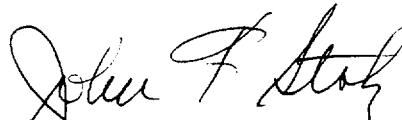
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated January 13, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 77, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 22, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 77

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
6-1	6-1
6-9	6-9
6-11	6-11
6-12	6-12
6-13	6-13
6-14	6-14
6-15	6-15

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The General Manager - Hope Creek Operations shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Senior Nuclear Shift Supervisor, or during his absence from the control room, a designated individual shall be responsible for the control room command function. A management directive to this effect, signed by the Chief Nuclear Officer and President - Nuclear Business Unit shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### 6.2.1 ONSITE AND OFFSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Hope Creek Generating Station Updated Final Safety Analysis Report and updated in accordance with 10 CFR 50.71(e).
- b. The General Manager - Hope Creek Operations shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Chief Nuclear Officer and President - Nuclear Business Unit shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

#### UNIT STAFF

6.2.2 The unit organization shall be subject to the following:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.2-1;

AUTHORITY

6.5.1.8 The SORC shall:

- a. Provide recommendations in writing to the General Manager - Hope Creek Operations indicating approval or disapproval of items considered under Specification 6.5.1.6 prior to their implementation.
- b. Provide written notification within 24 hours to the Vice President - Nuclear Operations and to the Director - Quality Assurance and Nuclear Safety Review of disagreement between the SORC and the General Manager - Hope Creek Operations; however, the General Manager - Hope Creek Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS AND REPORTS

6.5.1.9 The SORC shall maintain minutes of each SORC meeting, and copies shall be provided to the Vice President - Nuclear Operations, Director - Quality Assurance and Nuclear Safety Review and Manager - Nuclear Safety.

6.5.2 NUCLEAR SAFETY REVIEW AND AUDIT

FUNCTION

6.5.2.1 The Nuclear Safety Department shall function to provide the independent safety review program and audit of designated activities.

COMPOSITION

6.5.2.2 Nuclear Safety Department, under the cognizance of the Director - Quality Assurance and Nuclear Safety Review, shall consist of the Manager - Nuclear Safety, the Offsite Safety Review staff (OSR) and the Onsite Safety Review Group (SRG). The OSR staff and the SRG each consist of at least four dedicated, full-time engineers.

The Manager - Nuclear Safety and the Offsite Safety Review staff shall meet or exceed the qualifications described in Section 4.7 of ANS 3.1 - 1981 and shall be guided by the provisions for independent review described in Section 4.3 of ANSI N18.7-1976 (ANS 3.2).

The Offsite Safety Review staff shall generally possess experience and competence in the areas listed in Section 6.5.2.4.1. A system of qualified reviewers from other technical organizations shall be utilized to augment OSR staff expertise in the disciplines of Section 6.5.2.4.1, where appropriate. Such qualified reviewers shall meet the same qualification requirements as the Offsite Safety Review staff, and shall not have been involved with performance of the original work.

The Onsite Safety Review Group staff shall meet or exceed the qualifications described in Section 4.4 of ANS 3.1 - 1981.

- f. All REPORTABLE EVENTS.
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- h. Reports and meeting minutes of the SORC.

AUDITS

6.5.2.4.3 Audits of facility activities shall be performed under the cognizance of the OSR staff. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months;
- f. The Facility Security Plan and implementing procedures at least once per 12 months;
- g. Any other area of facility operation considered appropriate by the Director - Quality Assurance and Nuclear Safety Review or the Chief Nuclear Officer and President - Nuclear Business Unit.
- h. The facility Fire Protection Program and the implementing procedures at least once per 24 months;
- i. The fire protection and loss prevention program implementation at least once per 12 months utilizing either a qualified off-site licensee fire protection engineer(s) or an outside independent fire protection consultant. An outside independent fire protection consultant shall be utilized at least once per 36 months; and
- j. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- k. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;

1. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months; and,
- m. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

The above audits shall be conducted by the Quality Assurance Department or an independent consultant. Audit plans and final audit reports shall be reviewed by the OSR staff.

#### RECORDS AND REPORTS

6.5.2.4.4 Records of OSR activities shall be maintained. Reports of reviews and audits shall be prepared and distributed as indicated below:

- a. The results of reviews performed pursuant to Section 6.5.2.4.2 shall be reported to the Chief Nuclear Officer and President - Nuclear Business Unit at least monthly.
- b. Audit reports prepared pursuant to Specification 6.5.2.4.3 shall be forwarded by the auditing organization to the Chief Nuclear Officer and President - Nuclear Business Unit and to the management positions responsible for the areas audited (1) within 30 days after completion of the audit for those audits conducted by the Nuclear Quality Assurance Department, and (2) within 60 days after completion of the audit for those audits conducted by an independent consultant.

#### 6.5.2.5 ONSITE SAFETY REVIEW GROUP (SRG)

##### FUNCTION

6.5.2.5.1 The SRG shall function to provide: the review of plant design and operating experience for potential opportunities to improve plant safety; evaluation of plant operations and maintenance activities; and advice to management on the overall quality and safety of plant operations.

The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

##### RESPONSIBILITIES

6.5.2.5.2 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics, NRC issuances, industry advisories, and other appropriate sources of plant design and operating experience information which may indicate areas for improving plant safety.
- b. Review of selected facility features, equipment, and systems.
- c. Review of selected procedures and plant activities including maintenance, modification, operational problems, and operational analysis.

## ADMINISTRATIVE CONTROLS

- d. Surveillance of selected plant operations and maintenance activities to provide independent verification\* that they are performed correctly and that human errors are reduced to as low as reasonably achievable.

### AUTHORITY

6.5.2.6 The Nuclear Safety Department shall report to and advise the Chief Nuclear Officer and President - Nuclear Business Unit on those areas of responsibility specified in Sections 6.5.2.4 and 6.5.2.5.

### 6.5.3 TECHNICAL REVIEW AND CONTROL

#### ACTIVITIES

6.5.3.1 All programs and procedures required by Technical Specification 6.8 and changes thereto, and any other proposed procedures or changes thereto, which affect plant nuclear safety as determined by the General Manager - Hope Creek Operations, other than editorial or typographical changes, shall be reviewed as follows:

#### PROCEDURE RELATED DOCUMENTS

6.5.3.2 Procedures, Programs and changes thereto shall be reviewed as follows:

- a. Each newly created procedure, program or change thereto shall be independently reviewed by an individual knowledgeable in the area affected other than the individual who prepared the procedure, program or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Station Administrative procedures will be approved by the appropriate station Department Manager or by the General Manager - Hope Creek Operations. Each station Department Manager shall be responsible for a predesignated class of procedures. The General Manager - Hope Creek Operations shall approve Station Administrative Procedures, Security Plan implementing procedures and Emergency Plan implementing procedures.
- b. On-the-spot changes to procedures which clearly do not change the intent of the approved procedures shall be approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License. Revisions to procedures which may involve a change in intent of the approved procedures, shall be reviewed in accordance with Section 6.5.3.2.a above.
- c. Individuals responsible for reviews performed in accordance with item 6.5.3.2.a above shall be approved by the SORC Chairman and designated as a Station Qualified Reviewer. A system of Station Qualified Reviewers shall be maintained by the SORC Chairman. Each review shall include a written determination of whether or not additional

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\*Not responsible for sign-off function.

cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel. The Station Qualified Reviewers shall meet or exceed the qualifications described in Section 4.1 and 4.7 of ANS 3.1, 1981.

- d. If the Department Manager determines that the documents involved require a 10 CFR 50.59 safety evaluation, the documents shall be forwarded for SORC review and also to the OSR staff for an independent review to determine whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

#### NON-PROCEDURE RELATED DOCUMENTS

6.5.3.3 Tests or experiments, and changes to equipment or systems shall be forwarded for SORC review and also to the OSR staff for an independent review to determine whether or not an unreviewed safety question is involved. The results of OSR staff reviews will be provided to SORC. Recommendations for approval are made by SORC to the General Manager - Hope Creek Operations. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

#### RECORDS AND REPORTS

6.5.3.4 Written records of reviews performed in accordance with item 6.5.3.2a above, including recommendations for approval or disapproval, shall be maintained. Copies shall be provided to the General Manager - Hope Creek Operations, SORC, the OSR staff, and/or NRC as necessary when their reviews are required.

#### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified pursuant to the requirements of Section 50.72 to 10 CFR Part 50 and a report submittal pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC, and the results of this review shall be submitted to the OSR staff and the Chief Nuclear Officer and President - Nuclear Business Unit.

#### 6.7. SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Chief Nuclear Officer and President - Nuclear Business Unit and the Director - Quality Assurance and Nuclear Safety Review shall be notified within 24 hours.

- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the Director - Quality Assurance and Nuclear Safety Review and the Chief Nuclear Officer and President - Nuclear Business Unit within 30 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

#### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978.
- b. The applicable procedures required to implement the requirements of NUREG-0737 and supplements thereto.
- c. Refueling operations.
- d. Surveillance and test activities of safety-related equipment.
- e. Security Plan implementation.
- f. Emergency Plan implementation.
- g. Fire Protection Program implementation.
- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.
- j. Quality Assurance Program for effluent and environment monitoring.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved in accordance with specification 6.5.1.6 or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 On-the-Spot changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the unit management staff, at least one of whom holds a Senior Reactor Operator license on the unit affected; and



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated January 13, 1995, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Hope Creek Generating Station, Technical Specification (TS). The requested changes would revise the Administrative Controls Section (6.0) of the Technical Specifications (TS) for Hope Creek Generating Station to reflect organizational changes and resultant management title changes. As indicated on the marked-up pages in Attachment 2, PSE&G requests that:

- 1) Vice President and Chief Nuclear Officer will be replaced with Chief Nuclear Officer and President - Nuclear Business Unit in TS 6.1.2, 6.2.1.c, 6.5.2.4.3.g, 6.5.2.4.4.a, 6.5.2.4.4.b, 6.5.2.6, 6.6.1.b, 6.7.1.a, and 6.7.1.c.
- 2) Vice President and Chief Nuclear Officer will be replaced with Vice President - Nuclear Operations in TS 6.5.1.8.b, and 6.5.1.9.
- 3) In addition, General Manager - Quality Assurance and Nuclear Safety will be replaced with Director - Quality Assurance and Nuclear Safety Review in TS 6.5.1.8.b, 6.5.1.9, 6.5.2.2, 6.5.2.4.3.g, 6.7.1.a, 6.7.1.c.

2.0 EVALUATION

The licensee has decided to separate the Nuclear Department from the Electric Business Unit and establish the Nuclear Department as its own separate business unit. As a result of these organizational changes, several managements titles in the TS have changed. These title changes are administrative in nature. The staff finds the requested changes acceptable since they meet the appropriate acceptance criteria of Sections 13.1.1, 13.1.2-13.1.3, and 13.4 of the Standard Review Plan (NUREG-0800) and conform to Section 5 of the Improved Standard Technical Specifications.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State Official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. H. Moran

Date: August 22, 1995