

Mr. Leon R. Eliason
 Chief Nuclear Officer & President-
 Nuclear Business Unit
 Public Service Electric & Gas
 Company
 Post Office Box 236
 Hancocks Bridge, NJ 08038

September 15, 1997

SUBJECT: HOPE CREEK GENERATING STATION (TAC NO. M99191)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment No. 102 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 3, 1997.

This amendment makes changes to TS Table 3.6.3-1, "Primary Containment Isolation Valves." The changes to the TS table add four inboard isolation valves (relief valves) to four penetrations listed in that table. The revisions to TS Table 3.6.3-1 support plant modifications being made to address concerns raised in NRC Generic Letter 96-06, "Assurance of Equipment Operability and Containment Integrity During Design Basis Conditions." These modifications are currently planned to be completed during the upcoming refueling outage (RF07). The staff will respond separately to the licensee's response to Generic Letter 96-06.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
 David H. Jaffe, Senior Project Manager
 Project Directorate I-2
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Docket No. 50-354

- Enclosures: 1. Amendment No. 102 to License No. NPF-57
 2. Safety Evaluation

NRC FILE CENTER COPY

cc w/encls: See next page

<u>DISTRIBUTION</u>	BBoger	OGC	ACRS
Docket File	JStolz	GHill(2)	JLinville, RGN-I
PUBLIC	MO'Brien	CBerlinger	MHart
PDI-2 Reading	DJaffe	WBeckner	

*SEE PREVIOUS CONCURRENCE

OFFICE	PDI-2/PM	PDI-2/PM	PDI-2/PM	TSB	OGC
NAME	MHart	DJaffe:mh	MO'Brien	WBeckner	APH
DATE	9/10/97	8/29/97	8/29/97	1/197	9/10/97

OFFICE	EMEB/BC*	SCSB/BC	PDI-2/D
NAME	RWessman	CBerlinger	JStolz
DATE	08/20/97	8/27/97	9/10/97

Cons. not need from SPLB per 9/11 Telecon with Marsh

OFFICIAL RECORD COPY
 DOCUMENT NAME: HC99191.AMD

9709300209 970915
 PDR ADOCK 05000354
 PDR



1/1
 DFO/



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 15, 1997

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION (TAC NO. M99191)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment No. 102 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 3, 1997.

This amendment makes changes to TS Table 3.6.3-1, "Primary Containment Isolation Valves." The changes to the TS table add four inboard isolation valves (relief valves) to four penetrations listed in that table. The revisions to TS Table 3.6.3-1 support plant modifications being made to address concerns raised in NRC Generic Letter 96-06, "Assurance of Equipment Operability and Containment Integrity During Design Basis Conditions." These modifications are currently planned to be completed during the upcoming refueling outage (RF07). The staff will respond separately to the licensee's response to Generic Letter 96-06.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. H. Jaffe", with a long horizontal stroke extending to the right.

David H. Jaffe, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Amendment No. 102 to
License No. NPF-57
2. Safety Evaluation

cc w/encls: See next page

Mr. Leon R. Eliason
Public Service Electric & Gas
Company

Hope Creek Generating Station

cc:

Jeffrie J. Keenan, Esquire
Nuclear Business Unit - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

Manager - Joint Generation
Atlantic Energy
6801 Black Horse Pike
Egg Harbor Twp., NJ 08234-4130

Hope Creek Resident Inspector
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038

Richard Hartung
Electric Service Evaluation
Board of Regulatory Commissioners
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Mr. Louis Storz
Sr. Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

General Manager - Hope Creek Operations
Hope Creek Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. Elbert Simpson
Senior Vice President-
Nuclear Engineering
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Manager - Licensing and Regulation
Nuclear Business Unit - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. NPF-57

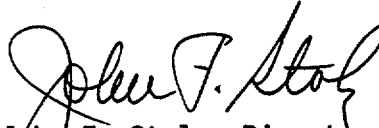
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated July 3, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 102, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 15, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-23
3/4 6-28

Insert

3/4 6-23
3/4 6-28

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>NOTE (S)</u>	<u>P&ID</u>
8. Group 8 - Torus Water Cleanup (TWC) System				
(a) TWC Suction Isolation Valves				M-53-1
Outside:				
HV-4680 (EE-V003)	P223	45	4	
HV-4681 (EE-V004)	P223	45	4	
(b) TWC Return Isolation Valves				M-53-1
Outside:				
HV-4652 (EE-V002)	P222	45	4	
HV-4679 (EE-V001)	P222	45	4	
9. Group 9 - Drywell Sumps				
(a) Drywell Floor Drain Sump Discharge Isolation Valves				M-61-1
Inside: HV-F003 (HB-V005)	P25	30	3	
Outside: HV-F004 (HB-V006)	P25	30	3	
Inside: 1HBPSV-11701	P25	NA	3	
(b) Drywell Equipment Drain Sump Discharge Isolation Valves				M-61-1
Inside: HV-F019 (HB-V045)	P26	30	3	
Outside: HV-F020 (HB-V046)	P26	30	3	
Inside: 1HBPSV-11702	P26	NA	3	
10. Group 10 - Drywell Coolers				
(a) Chilled Water to Drywell Coolers Isolation Valves				M-87-1
Inside:				
Loop A: HV-9531B1 (GB-V081)	P8B	60	3	
Loop B: HV-9531B3 (GB-V083)	P38A	60	3	

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>NOTE (S)</u>	<u>P&ID</u>
16. Group 16 - Reactor Auxiliaries Cooling System (RACS)				
(a) RACS Supply Isolation Valves				
Inside: HV-2554 (ED-V020)	P29	45	3	M-13-1
Outside: HV-2553 (ED-V019)	P29	45	3	
Inside: 1EDPSV-11699	P29	NA	3	
(b) RACS Return Isolation Valves				
Inside: HV-2556 (ED-V022)	P30	45	3	M-13-1
Outside: HV-2555 (ED-V021)	P30	45	3	
Inside: 1EDPSV-11700	P30	NA	3	
17. Group 17 - Traversing In-core Probe (TIP) System				
(a) TIP Probe Guide Tube Isolation Valves				
Outside:				
SV-J004A-1 (SE-V026)	P34A	15	3	M-59-1
SV-J004A-2 (SE-V027)	P34B	15	3	
SV-J004A-3 (SE-V028)	P34C	15	3	
SV-J004A-4 (SE-V029)	P34D	15	3	
SV-J004A-5 (SE-V030)	P34E	15	3	
(b) TIP Purge System Isolation Valve				
Outside:				
HV-5161 (SE-V004)	P34F	15	3	M-59-1
18. Group 18 - Reactor Coolant Pressure Boundary (RCPB) Leakage Detection System				
(a) Drywell Leak Detection Radiation Monitoring System (DLD-RMS) Inlet Isolation Valves				
Outside:				
HV-5018 (SK-V005)	J8C	45	3	M-25-1
HV-4953 (SK-V006)	J8C	45	3	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated July 3, 1997, the Public Service Electric & Gas Company (PSE&G, the licensee) submitted a request for changes to the Hope Creek Generating Station (HCGS), Technical Specifications (TSs). The requested changes would revise TS Table 3.6.3-1, "Primary Containment Isolation Valves." The changes to the TS table would add four inboard isolation valves (relief valves) to four penetrations listed in that table. The revisions to TS Table 3.6.3-1 would support plant modifications being made to address concerns raised in NRC Generic Letter (GL) 96-06, "Assurance of Equipment Operability and Containment Integrity During Design Basis Conditions." These modifications are currently planned to be completed during the upcoming refueling outage (RF07).

2.0 BACKGROUND

GL 96-06 was issued on September 30, 1996. Among the safety-significant issues identified was that thermally induced overpressurization of isolated water-filled piping sections in containment could jeopardize the ability of accident mitigating systems to perform their safety functions and could also lead to a breach of containment integrity via bypass leakage.

By letter dated January 27, 1997, PSE&G made the 120-day response to GL 96-06 for the HCGS. In this report, PSE&G identified five containment penetrations that were susceptible to overpressurization associated with thermal expansion of the fluid in the piping. PSE&G also stated that the overpressurization concern would be addressed by making a design modification to the affected penetrations. The modification would install pressure relief devices on penetrations P10, P25, P26, P29, and P30, with installation to be completed by the end of the next refueling outage (RF07).

The July 3, 1997, licensee submittal proposed changes to TS Table 3.6.3-1 that add four inboard isolation valves (relief valves) to containment penetrations P25, P26, P29, and P30. Approval of the proposed TS changes would enable the design modifications, as stated in PSE&G's January 27, 1997, letter, to be installed.

To address GL 96-06 overpressurization concerns, containment penetration P10 would have a relief valve added during the same modification, but because, in this case, the added relief valve is not considered a containment isolation valve, there is no TS change associated with the addition of that valve. The P10 penetration, inside containment isolation valve was reclassified to no longer be a containment isolation valve in Amendment 93 to the HCGS License, issued February 22, 1996. Because there is no inboard primary containment isolation valve for P10, the added relief valve is not considered a primary containment isolation valve either.

3.0 EVALUATION

The licensee states that the proposed spring loaded, water (incompressible fluid) relief valves would be designed to PSE&G Piping Specification P-500 and ASME Codes applicable to the existing penetrations and isolation valves, and would also be required to meet the same surveillance requirements already specified for other relief valves that are containment isolation valves. The valves would be designated as safety-related and Seismic Category I components, except for the discharge piping, which would be non-safety related, seismically analyzed and designed to the requirements of ANSI B31.1. The staff finds this acceptable.

The licensee states that the design of the proposed relief valves would maintain containment integrity in the event of a loss-of-coolant accident inside containment. The licensee also states that proposed relief valve setpoints would be selected to ensure that the ASME Boiler and Pressure Vessel, Section III, pressure ratings will not be exceeded for the existing piping and isolation valves in the affected penetrations, and that the valve bodies and internals for the proposed relief valves would be qualified for the maximum pressure and temperature ratings specified in the ASME Boiler and Pressure Vessel Code. The staff has determined that the proposed relief valves are designed to the applicable codes and maintain containment integrity. The staff has also determined that the proposed relief valves do not affect the ability of the affected penetrations to withstand a single failure.

The staff has concluded that the installation of the proposed relief valves to address overpressurization concerns raised in GL 96-06 would not keep the systems, structures and components from performing their safety functions, and that the modification and the TS changes associated with the modification are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State Official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 43375). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Hart

Date: September 15, 1997