

November 6, 1997

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION (TAC NO. M99445)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment No. 110 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 20, 1997.

This amendment changes the Technical Specifications (TSs) to provide for: 1) the relocation of suppression chamber volume references in Limiting Condition for Operation (LCO) 3.5.3 to the Hope Creek (HC) Updated Final Safety Analysis Report (UFSAR) and TS Bases as appropriate; 2) the revision of the suppression chamber volume currently listed in LCO 3.5.3.b; 3) the relocation of the suppression chamber volume references in LCO 3.6.2.1.a.1 to the UFSAR and TS Bases; and 4) the revision to the suppression chamber volume reference in TS 5.2.1 to reference the TS Bases section where this information will reside.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
David H. Jaffe, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket No. 50-354

- Enclosures: 1. Amendment No. 110 to License No. NPF-57
2. Safety Evaluation

cc w/encls: See next page

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State comment discussed with J. Moore on 11/4/97. No problem noted.

OFFICE	PDI-2/RM	PDI-2/LA	TSB/BC	SCSB/BC	OGC	PDI-1/D
NAME	DJaffe:cw	MO'Brien	WBeckner	CBerlinger	JStolz	JStolz
DATE	10/10/97	10/10/97	10/17/97	10/14/97	10/29/97	11/15/97

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Collins
10/16/97*

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DOCUMENT NAME: HC99445.AMD

Licensee App. C change must be received before issuance



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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 6, 1997

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
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Post Office Box 236
Hancocks Bridge, NJ 08038

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A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. H. Jaffe", written over a circular stamp or mark.

David H. Jaffe, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Amendment No. 110 to
License No. NPF-57
2. Safety Evaluation

cc w/encls: See next page

Mr. Leon R. Eliason
Public Service Electric & Gas
Company

Hope Creek Generating Station

cc:

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Hancocks Bridge, NJ 08038

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U.S. Nuclear Regulatory Commission
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Hancocks Bridge, NJ 08038

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c/o Mary O. Henderson, Clerk
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Hancocks Bridge, NJ 08038

General Manager - Hope Creek Operations
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Hancocks Bridge, NJ 08038

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Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated August 20, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 110, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

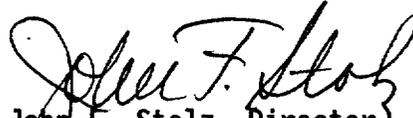
In addition, paragraph 2.C.(14) to Facility Operating License No. NPF-57 is amended as follows:

(14) Additional Conditions

The Additional Conditions contained in Appendix C, as revised through Amendment No. 110, are hereby incorporated into this license. Public Service Electric and Gas Company shall operate the facility in accordance with the Additional Conditions.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days. Implementation of this amendment shall include the relocation of the suppression chamber water volume, as contained in Technical Specifications 3.5.3.a, 3.5.3.b, 3.6.2.1.a.1 and 5.2.1 to the Updated Final Safety Analysis Report, as described in the licensee's application dated August 20, 1997, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

- Attachments: 1. Page 1 to Appendix C of License* No. NPF-57
2. Changes to the Technical Specifications

Date of Issuance: November 6, 1997

*Page 1 to Appendix C is attached, for convenience, for the composite license to reflect this change.

ATTACHMENT TO LICENSE AMENDMENT NO. 110

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

1. Insert Appendix C, Page 1
2. Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 5-8
3/4 6-12
5-1

Insert

3/4 5-8
3/4 6-12
5-1

APPENDIX C

ADDITIONAL CONDITIONS
OPERATING LICENSE NO. DPR-57

Public Service Electric and Gas Company and Atlantic City Electric Company shall comply with the following conditions on the schedules noted below:

Amendment Number	Additional Condition	Implementation Date
97	The licensee is authorized to relocate certain Technical Specification requirements to licensee-controlled documents. Implementation of this amendment shall include the relocation of these technical specification requirements to the appropriate documents, as described in the licensee's application dated January 11, 1996, as supplemented by letters dated February 26, May 22, June 27, July 12, December 23, 1996, and March 17, 1997, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from March 21, 1997.
103	The licensee shall relocate the list of "Motor Operated Valves - Thermal Overload Protection (BYPASSED)" from the Technical Specifications (Table 3.8.4.2-1) to the Updated Final Safety Analysis Report, as described in the licensee's application dated July 7, 1997, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from September 16, 1997
105	The licensee shall use the Banked Pattern Withdrawal System or an improved version such as the Reduced Notch Worth Procedure as described in the licensee's application dated June 19, 1997, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from September 30, 1997
110	The licensee shall relocate the suppression chamber water volume, as contained in Technical Specifications 3.5.3.a, 3.5.3.b, 3.6.2.1.a.1 and 5.2.1 to the Updated Final Safety Analysis Report, as described in the licensee's application dated August 20, 1997, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from November 6, 1997

EMERGENCY CORE COOLING SYSTEMS

3/4.5.3 SUPPRESSION CHAMBER

LIMITING CONDITION FOR OPERATION
=====

3.5.3 The suppression chamber shall be OPERABLE:

- a. In OPERATIONAL CONDITION 1, 2 and 3 with an indicated water level of at least 74.5".
- b. In OPERATIONAL CONDITION 4 and 5* with an indicated water level of at least 5.0" except that the suppression chamber level may be less than the limit or may be drained provided that:
 - 1. No operations are performed that have a potential for draining the reactor vessel,
 - 2. The reactor mode switch is locked in the Shutdown or Refuel position,
 - 3. The condensate storage tank contains at least 135,000 available gallons of water, and
 - 4. The core spray system is OPERABLE per Specification 3.5.2 with an OPERABLE flow path capable of taking suction from the condensate storage tank and transferring the water through the spray sparger to the reactor vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4 and 5*.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2 or 3 with the suppression chamber water level less than the above limit, restore the water level to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 4 or 5* with the suppression chamber water level less than the above limit or drained and the above required conditions not satisfied, suspend CORE ALTERATIONS and all operations that have a potential for draining the reactor vessel and lock the reactor mode switch in the Shutdown position. Establish SECONDARY CONTAINMENT INTEGRITY within 8 hours.

*The suppression chamber is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded or being flooded from the suppression pool, the spent fuel pool gates are removed when the cavity is flooded, and the water level is maintained within the limits of Specifications 3.9.8 and 3.9.9.

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION SYSTEMS

SUPPRESSION CHAMBER

LIMITING CONDITION FOR OPERATION
=====

3.6.2.1 The suppression chamber shall be OPERABLE with:

- a. The pool water:
 - 1. With an indicated water level between 74.5" and 78.5" and a
 - 2. Maximum average temperature of 95°F during OPERATIONAL CONDITION 1 or 2, except that the maximum average temperature may be permitted to increase to:
 - a) 105°F during testing which adds heat to the suppression chamber.
 - b) 110°F with THERMAL POWER less than or equal to 1% of RATED THERMAL POWER.
 - 3. Maximum average temperature of 95°F during OPERATIONAL CONDITION 3, except that the maximum average temperature may be permitted to increase to 120°F with the main steam line isolation valves closed following a scram.
- b. A total leakage between the suppression chamber and drywell of less than the equivalent leakage through a 1-inch diameter orifice at a differential pressure of 0.80 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With the suppression chamber water level outside the above limits, restore the water level to within the limits within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the suppression chamber average water temperature greater than 95°F and THERMAL POWER greater than 1% of RATED THERMAL POWER, restore the average temperature to less than or equal to 95°F within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours, except, as permitted above:
 - 1. With the suppression chamber average water temperature greater than 105°F during testing which adds heat to the suppression chamber, stop all testing which adds heat to the suppression chamber and restore the average temperature to less than 95° within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - 2. With the suppression chamber average water temperature greater than 110°F, place the reactor mode switch in the Shutdown position and operate at least one residual heat removal loop in the suppression pool cooling mode.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated August 20, 1997, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Hope Creek Generating Station, Technical Specification (TSs). The requested amendment would change the TSs to provide for: 1) the relocation of suppression chamber volume references in Limiting Condition for Operation (LCO) 3.5.3 to the Hope Creek (HC) Updated Final Safety Analysis Report (UFSAR) and TS Bases as appropriate; 2) the revision of the suppression chamber volume currently listed in LCO 3.5.3.b; 3) the relocation of the suppression chamber volume references in LCO 3.6.2.1.a.1 to the UFSAR and TS Bases; and 4) the revision to the suppression chamber volume reference in TS 5.2.1 to reference the TS Bases section where this information will reside.

2.0 DISCUSSION

With regard to the proposed relocation of the Torus water volume requirements, the licensee has proposed the relocation of the Torus water volume, specified in TSs 3.5.3, 3.6.2.1.a.1 and 5.2.1 to the UFSAR. Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to state TSs to be included as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

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The regulation does set forth four criteria to be used in determining whether a limiting condition for operation is required to be included in the TSs, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. Existing TS requirements which fall within or satisfy any of the criteria must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

The application also addresses a proposed change to the existing Torus water volume as specified in TS 3.5.3.b, and minor wording changes to the TS that are necessitated by removal of the Torus water volume requirements. In addition, the licensee has proposed a change to TS 5.2.1 that would reference the TS Bases to identify the location of the requirements for Torus water volume.

3.0 EVALUATION

The August 20, 1997 application seeks to relocate the Torus Water volume from TS 3.5.3, 3.6.2.1.a.1 and 5.2.1 to the UFSAR. The Torus, also referred to as the suppression chamber, is a torus-shaped tank located below, and connected to, the drywell that contains the reactor pressure vessel and connected piping. During normal operation, the Torus absorbs heat from safety-relief valves and other sources. Under Loss-of-Coolant accident conditions, the Torus suppresses drywell pressure and provides a source of water for long-term core cooling. The staff's review of the proposed changes determined that the relocation of the Torus water volume does not eliminate the requirements for the licensee to ensure that the Torus water volume is properly maintained. Once the Torus water volume is relocated from the TSs to the UFSAR, the licensee must continue to evaluate any changes to it in accordance with 10 CFR 50.59. Should the licensee's determination conclude that an unreviewed safety question is involved, due to either (1) an increase in the probability or consequences of accidents or malfunctions of equipment important to safety, (2) the creation of a possibility for an accident or malfunction of a different type than any evaluated previously, or (3) a reduction in the margin of safety, NRC approval and a license amendment would be required prior to implementation of the change. NRC inspection and enforcement programs also enable the staff to monitor facility changes and licensee adherence to updated

final safety analysis report commitments and to take any remedial action that may be appropriate. Furthermore, there is a direct relationship between torus water volume (inventory) and torus water level. The TS will continue to specify a minimum water level limit for ECCS purposes and minimum/maximum level limits for containment functional purposes. This will provide assurance that the licensee will bring the plant to a safe condition in the event that the water volume (inventory) is outside acceptable limits for the operating condition.

The staff's review has concluded that 10 CFR 50.36 does not require the Torus water volume to be retained in the TSs. Torus water level, alone, satisfies the requirements of *Criterion 2* of 10 CFR 50.36 as discussed above. Moreover, Torus water level is an observable "process variable" which is available to the plant operators while Torus water volume must be calculated; thus, Torus water level is more appropriate as one observable measure of Torus operability. The Torus water level will continue to be specified in TS 3.5.3a and b, and 3.6.2.1.a.1. However, the staff determined that the inclusion of the Torus water volume is an operational detail related to the licensee's safety analyses which are adequately controlled by the requirements of 10 CFR 50.59. Therefore, the continued processing of license amendments related to revisions of the affected TS 3.5.3, 3.6.2.1.a.1 and 5.2.1 where the revisions to those requirements do not involve an unreviewed safety question under 10 CFR 50.59, would afford no significant benefit with regard to protecting the public health and safety. Accordingly, the relocation of the Torus Water volume from TS 3.5.3, 3.6.2.1.a.1 and 5.2.1 to the UFSAR is acceptable.

With regard to the Torus water volume specified in TS 3.5.3.b, for OPERATIONAL CONDITIONS 4 and 5 (COLD SHUTDOWN AND REFUELING, respectively), the licensee is proposing that the minimum specified Torus water volume be reduced from 57,390 cubic feet to 57,232 cubic feet, a reduction of 0.3%. The proposed Torus water volume reduction is needed to support installation of improved Torus suction strainers in accordance with NRC's Generic Letter 96-03, "Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling Water Reactors", May 6, 1996. As indicated in TS Bases 3/4.5.3, the minimum Torus water volume under OPERATIONAL CONDITIONS 4 and 5 is based upon net positive suction head (NPSH), recirculation volume, vortex prevention and a safety margin. The licensee has performed a qualitative assessment of Emergency Core Cooling System performance under OPERATIONAL CONDITIONS 4 and 5 and has concluded that a reduction of 0.3% Torus water volume will have no detrimental effects on safety system performance. The NRC Staff agrees with the licensee's assessment, considering the relatively small change in Torus Water volume, and finds the change to be acceptable.

The licensee has also proposed minor wording changes to TS 3.5.3 and 3.6.2.1.a.1 that are necessitated by removal of the Torus water volume requirements. For example, the phrase "at least" would be inserted in the text of TS 3.5.3, before the indicated Torus water level, to retain the meaning of the TS requirement. These changes are minor, are of an administrative and clarifying nature, and do not effect the TS requirements. Accordingly, these proposed changes to the TS are acceptable.

Finally, the licensee has proposed an addition to TS 5.2.1 that would reference the TS Bases to identify the location of the requirements for Torus water volume. At the current time, TS Bases 3/4.6.2 contains the applicable Torus Water volume, which is not changed by the proposed TS. This reference is helpful to the TS user, is administrative in nature, and does not effect any safety requirement in the TS. Based upon the above, the proposed change to TS 5.2.1 is acceptable.

The relocation of the Torus water level to the UFSAR will be undertaken in accordance with a license condition contained in Appendix C to the Facility Operating License, NPF-57, for Hope Creek Generating Station. By letter dated October 28, 1997, the licensee indicated their agreement with the subject license condition.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State Official was notified of the proposed issuance of the amendment. In an October 24, 1997 telephone conversation, the State official, Mr. R. Pinney noted that, while the Torus volume had been removed from TS 3.6.2.1, TS 4.6.2.1.a still requires that the Torus volume be verified to be within specified limits every 24 hours. During a subsequent inspection, it was determined that the licensee is required, by procedure, to determine that the Torus level is within specified limits every 24 hours. A note appears at the bottom of the procedure data sheet which correlates acceptable Torus level to an acceptable Torus volume. Thus, the licensee can comply with TS 3.6.2.1, as amended, and TS 4.6.2.1.a.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 50010). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. H. Jaffe

Date: November 6, 1997