50-354

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 12, 1998

Mr. Harold W. Keiser
Chief Nuclear Officer & President
Nuclear Business Unit
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: CORRECTION TO AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-57, HOPE CREEK GENERATING STATION (TAC NO. MA3847)

Dear Mr. Keiser:

On June 5, 1989, the Nuclear Regulatory Commission issued Amendment No. 26 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment increased the surveillance test intervals and allowable out-of-service times for the Reactor Protection System, as requested by an application from Public Service Electric & Gas Company (PSE&G) dated February 6, 1989, as supplemented May 4, 1989.

In a letter dated October 8, 1998, PSE&G identified two typographical errors on Technical Specification (TS) Table 4.3.1.1-1, "Reactor Protection System Instrumentation Surveillance Requirements," that were introduced when TS Amendment No. 26 was issued by the NRC. Both errors involve TS Table 4.3.1.1-1, Function 8.b, which specifies the surveillance requirements for the Scram Discharge Volume Water Level - High, Level Transmitter/Trip Unit.

The first error was the inadvertent omission of a left hand bracket in the "Channel Functional Test" column of the table. PSE&G's supplement dated May 4, 1989, indicated that the "Channel Functional Test" column for Function 8.b should be changed from "M" (i.e., monthly) to "$Q^{(k)}$" (i.e., quarterly with a reference to note "k"). Amendment No. 26 incorrectly changed this column to read "$Q^{k)}$."

The second error was the inadvertent deletion of Operational Condition 2 in the "Operational Conditions for which Surveillance Required" column of the table. PSE&G's application dated February 6, 1989, and the supplement dated May 4, 1989, did not request any changes to this column. Prior to Amendment No. 26, this column in TS Table 4.3.1.1-1 for Function 8.b read "1, 2, $5^{(i)}$." Amendment No. 26 incorrectly changed this column to read "1, $5^{(i)}$."

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Since the two typographical errors introduced by Amendment No. 26 were not addressed in the notice to the public and were not addressed by the NRC staff review, the corrections of these typographical errors are not considered "changes" to the Hope Creek Generating Station TSs pursuant to 10 CFR 50.92. Accordingly, corrections to the affected TS page are enclosed with this letter.

Sincerely,

original signed by:
Richard B. Ennis, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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TABLE 4.3.1.1-1 (Continued)
REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
8. Scram Discharge Volume Water Level - High				
a. Float Switch	NA	Q	R	1, 2, 5(j)
b. Level Transmitter/Trip Unit	S	Q(k)	R	1, 2, 5(j)
9. Turbine Stop Valve - Closure	NA	Q	R	1
10. Turbine Control Valve Fast Closure Valve Trip System Oil Pressure - Low	NA	Q	R	1
11. Reactor Mode Switch Shutdown Position	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram	NA	W	NA	1, 2, 3, 4, 5

(a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
(b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decades during each startup after entering OPERATIONAL CONDITION 2 and the IRM and APRM channels shall be determined to overlap for at least 1/2 decades during each controlled shutdown, if not performed within the previous 7 days.
(c) Within 24 hours prior to startup, if not performed within the previous 7 days.
(d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER ≥ 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER. Any APRM channel gain adjustment made in compliance with Specification 3.2.2 shall not be included in determining the absolute difference.
(e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.
(f) The LPRMs shall be calibrated at least once per 1000 effective full power hours (EFPH) using the TIP system.
(g) Verify measured core flow (total core flow) to be greater than or equal to established core flow at the existing recirculation loop flow (APRM % flow).
(h) This calibration shall consist of verifying the 6 ± 0.6 second simulated thermal power time constant.
(i) This item intentionally blank
(j) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
(k) Verify the tripset point of the trip unit at least once per 92 days.