May 25, 1995

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: CHANGES TO TECHNICAL SPECIFICATIONS FOR LOOSE PARTS DETECTION SYSTEM REQUIREMENTS AND RELOCATING THE REQUIREMENTS IN THE UPDATED FINAL SAFETY ANALYSIS REPORT, HOPE CREEK GENERATING STATION (TAC NO. M90329)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 30, 1994.

This amendment removes from the TSs those specifications associated with the Loose-Part Detection System (LPDS). Specifically, TS 3.3.7.9, Loose-Part Detection System, Surveillance Requirement 4.3.7.9, associated Bases and the index are revised to indicate the LPDS requirements have been removed. The LPDS requirements will subsequently be relocated in the Updated Final Safety Analysis Report.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
original signed by
David Moran, Acting Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Amendment No. 73 to License No. NPF-57
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION

Docket File	JStolz	GHill(2)	OC/LFDCB
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JZwolinski	OPA		

*See previous concurrence

OFFICE	PDI-2/LA	PDI-2/PM	SRXB	OGC	PDI-2/D
NAME	MO'Brien	DMoran:mw	RJones		JStolz
DATE	5/23/95	5/23/95	5/23/95	5/08/95	5/24/95

OFFICIAL RECORD COPY FILENAME: HC90329.AMD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated August 30, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 73 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 25, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
ix	ix
xviii	xviii
3/4 3-90	3/4 3-90
B 3/4 3-6	B 3/4 3-6

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

BASES

PAGE 3/4 3-90 IS DELETED

MONITORING INSTRUMENTATION (Continued)

3/4.3.7.8

The material originally contained in Section 3/4.3.7.8 was deleted with the issuance of the Full Power license. However, to maintain numerical continuity between the succeeding sections and existing station procedural references to those Technical Specifications Sections 3/4.3.7.8 has been intentionally left blank.

3/4.3.7.9 LOOSE-PART DETECTION SYSTEM

THIS SECTION DELETED

3/4.3.7.10 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

The radioactive liquid effluent monitoring instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in liquid effluents during actual or potential releases of liquid effluents. The alarm/trip points for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

3/4.3.7.11 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent monitoring instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The alarm/trip setpoints for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM. This will ensure the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. This instrumentation also includes provisions for monitoring and controlling the concentrations of potentially explosive gas mixtures in the main condenser offgas treatment system. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated August 30, 1994, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Hope Creek Generating Station, Technical Specification (TSs). The requested changes will relocate TS 3.3.7.9, Loose Parts Detection System (LPDS), Surveillance Requirement 4.3.7.9, and associated Bases from the TSs to the Updated Final Safety Analysis Report (UFSAR). The TSs index will also be revised by removing the reference to LPDS. Removal of the LPDS requirements from the TSs is consistent with the NRC Final Policy Statement on TS Improvements (58 FR 39132). As a result of the proposed changes, subsequent revisions to the LPDS requirements can be processed in accordance with 10 CFR 50.59 as opposed to 10 CFR 50.90.

2.0 EVALUATION

The Commission's regulatory requirements related to the content of the TSs are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The LPDS identifies the existence of possible loose parts in the reactor coolant system. Early detection can provide operators time to take corrective actions and avoid or mitigate damage to or malfunctions of primary components. However, as discussed in the Final Policy Statement on TS improvements, the LPDS does not function to detect significant abnormal degradation of the reactor coolant pressure boundary, the LPDS does not serve as an active design feature for establishing initial conditions for mitigation of design basis accidents or transients. The licensee has proposed to relocate these provisions to the UFSAR such that future changes to the operation and surveillance of the LPDS could be changed under 10 CFR 50.59.

Accordingly, the staff has concluded that the requirements for the LPDS do not meet the TS criteria in the Final Policy Statement on TS improvements. The limiting conditions for operation and surveillance requirements for the LPDS were removed from the standard technical specifications.

In conclusion, this specific instrumentation requirement is not required to be in the TSs under 10 CFR 50.36 or Section 182a of the Atomic Energy Act, as amended, and is not required to obviate the possibility of an abnormal situation event giving rise to an immediate threat to the public health and safety. Further, LPDS does not fall within any of the four criteria set forth in the Commission's Final Policy Statement discussed above. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.59 to address future changes to these requirements. Accordingly, the staff has concluded that these requirements may be relocated from the TSs to the licensee's UFSAR.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State Official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 16197). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Moran

Date: May 25, 1995