

February 1, 1990

Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger

SUBJECT: ALLOWANCE TO CONTINUE OPERATION WITH AN OUT OF SERVICE VALVE
POSITION INDICATION ON A PRIMARY CONTAINMENT ISOLATION VALVE IF THE
PENETRATION IS ISOLABLE (TAC NO. 69581)

Re: HOPE CREEK GENERATING STATION

The Commission has issued the enclosed Amendment No. 36 to Facility Operating
License No. NPF-57 for the Hope Creek Generating Station. This amendment
consists of changes to the Technical Specifications (TSs) in response to your
application dated September 28, 1988 and supplemented on July 21, 1989.

This amendment revises the Technical Specifications to avert the currently
required plant shutdown in the event that position indication instrumentation
for a primary containment isolation valve in an otherwise isolated penetration
is declared inoperable.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Clyde Shiraki, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 36 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

DFo1
1/1

DISTRIBUTION w/enclosures:

Docket File	MO'Brien (2)	Wanda Jones	SVarga
NRC PDR	OGC	JCalvo	BBoger
Local PDR	DHagan	Tech Branch	JDyer
PDI-2 Reading	EJordan	ACRS (10)	JLinville
WButler	BGrimes	GPA/PA	
CShiraki(3)/SBrown	GHill (4)	Rita Jaques, ARM/LFMB	

[69581 AMEND]

P 9002160225 900201
PDR ADDOK 05000354
PDC

PDI-2/PM
MO'Brien
1/9/89

PDI-2/PM
CShiraki:mj
01/05/89

PDI-2/D
WButler
2/1/89

OGC
BMB
1/10/89

SIG
SNewberry
1/31/89

LB



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

February 1, 1990

Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: ALLOWANCE TO CONTINUE OPERATION WITH AN OUT OF SERVICE VALVE
POSITION INDICATION ON A PRIMARY CONTAINMENT ISOLATION VALVE IF THE
PENETRATION IS ISOLABLE (TAC NO. 69581)

Re: HOPE CREEK GENERATING STATION

The Commission has issued the enclosed Amendment No. 36 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated September 28, 1988 and supplemented on July 21, 1989.

This amendment revises the Technical Specifications to avert the currently required plant shutdown in the event that position indication instrumentation for a primary containment isolation valve in an otherwise isolated penetration is declared inoperable.

A copy of our safety evaluation is also enclosed. Notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Clyde Shiraki".

Clyde Shiraki, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 36 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas Co.

Hope Creek Generating Station

cc:

M. J. Wetterhahn, Esquire
Conner & Wetterhahn
Suite 1050
1747 Pennsylvania Avenue
Washington, D.C. 20006

Mr. Scott B. Ungerer, Manager
Joint Generation Projects Department
Atlantic Electric Company
Post Office Box 1500
Pleasantville, New Jersey 08237

R. Fryling, Jr., Esquire
Law Department - Tower 5E
80 Park Place
Newark, New Jersey 07101

Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 241
Hancocks Bridge, New Jersey 08038

Mr. S. LaBruna
Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Mr. J. J. Hagan
General Manager - Hope Creek Operations
Hope Creek Generating Station
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Mr. B. A. Preston, Manager
Licensing and Regulation
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Dr. Jill Lipoti, Ph.D
New Jersey Department of Environmental
Protection
Radiation Protection Program
State of New Jersey
CN 415
Trenton, New Jersey 08625



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated September 28, 1988 and supplemented on July 21, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 36, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 1, 1990

*PDI-2/PA
CShiraki
1/19/89*

*PDI-2/PM
CShiraki:mj
01/05/89*

*PDI-2/D
WButler
2/1/89*

*OGC
Bms
1/10/89
90*

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: **February 1, 1990**

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Overleaf page provided to maintain document completeness.*

Remove

3/4 3-85

3/4 3-86

Insert

3/4 3-85*

3/4 3-86

HOPE CREEK

**TABLE 3.3.7.5-1
ACCIDENT MONITORING INSTRUMENTATION**

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1,2,3	80
2. Reactor Vessel Water Level	2	1	1,2,3	80
3. Suppression Chamber Water Level	2	1	1,2,3	80
4. Suppression Chamber Water Temperature*	2	2	1,2,3	80(a)
5. Suppression Chamber Pressure	2	1	1,2,3	80
6. Drywell Pressure	2	1	1,2,3	80
7. Drywell Air Temperature	2	1	1,2,3	80
8. Primary Containment Hydrogen/Oxygen Concentration Analyzer and Monitor	2	1	1,2,3	80
9. Safety/Relief Valve Position Indicators ^(c)	2/valve**	1/valve**	1,2,3	80
10. Drywell Atmosphere Post-Accident Radiation Monitor	2	1	1,2,3	81
11. North Plant Vent Radiation Monitor#	1	1	1,2,3	81
12. South Plant Vent Radiation Monitor#	1	1	1,2,3	81
13. FRVS Vent Radiation Monitor#	1	1	1,2,3	81
14. Primary Containment Isolation Valve Position Indication ^(b)	2/valve	1/valve	1,2,3	82

#High range noble gas monitors.

*Average bulk pool temperature.

**Acoustic monitoring and tall pipe temperature.

(a) Suppression chamber water temperature instrumentation must satisfy the availability requirements specified in Specification 3.6.2.1.

(b) One channel consists of the open limit switch, and the other channel consists of the closed limit switch.

(c) The acoustic monitor for FO13H SRV may be inoperable until September 21, 1987 or until the first forced outage of sufficient duration to effect repair prior to that date without applying the shutdown requirement of ACTION 80(a).

3/4 3-85

Amendment No. 5
Effective Date: June 4, 1987
JUN 17 1987

Table 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION
ACTION STATEMENTS

ACTION 80 -

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

ACTION 81 - With the number of OPERABLE accident monitoring instrumentation channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or:

- a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
- b. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

ACTION 82 -

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods. If the affected penetration is not isolated by either (i) a closed manual valve, (ii) a blind flange, or (iii) a deactivated automatic valve located outside primary containment, restore the inoperable channel(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods. If the affected penetration is not isolated by either (i) a closed manual valve, (ii) a blind flange, or (iii) a deactivated automatic valve located outside primary containment, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO.36 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated September 28, 1988 and supplemented on July 21, 1989, Public Service Electric & Gas Company requested an amendment to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. The proposed amendment would revise Technical Specification Table 3.3.7.5-1 to allow continued plant operation with a valve position indication out of service on a primary containment isolation valve if the penetration is isolable by a blind flange, a manual valve, or a deactivated automatic valve that is located outside the containment.

2.0 EVALUATION

Technical Specification 3.6.3, ACTION a. permits primary containment isolation valves to remain inoperable provided that another isolation valve in the affected penetration is OPERABLE, and requires that within 4 hours, if the inoperable valve cannot be restored to OPERABLE status the penetration must be isolated by either a deactivated automatic valve, a closed manual valve or a blind flange. If the penetration is isolated, operation may continue.

However, if, in addition to the inoperable primary containment isolation valve, a position indication instrument for the subject valve or another primary containment isolation valve in the same penetration becomes inoperable and cannot be restored to OPERABLE status, ACTION 82 a. or 82 b. for Technical Specification Table 3.3.7.5-1 would require a plant shutdown within either 30 or 7 days, respectively (depending on whether either or both position indication channels are inoperable.) But, if power has been removed from an automatic valve located outside the containment or a manual valve has been shut or a blind flange installed to isolate the penetration, the penetration is isolated, containment integrity is ensured and position indication of either primary containment isolation valve serves no function.

Restricting deactivation of an automatic valve to one which is located outside the containment ensures that if the INOPERABLE position indication is on the deactivated valve and if the deactivated valve is reopened as allowed by the note of specification 3.6.3, the final, shut position of the valve may be verified locally.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (53 FR 46158) on November 16, 1988. The supplemental information dated July 21, 1989 contained substantive changes to the original submittal so the no significant hazards consideration was renoticed in the Federal Register (54 FR 46157) on November 1, 1989. The staff also consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: C. Y. Shiraki

Dated: February 1, 1990