December 17, 1991

Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: MINIMUM CRITICAL POWER RATIO AMENDMENT, HOPE CREEK GENERATING STATION (TAC NO. M81928)

The Commission has issued the enclosed Amendment No. 47 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 17, 1991.

This amendment revised Section 3/4.2.3, MINIMUM CRITICAL POWER RATIO of the TSs. Specifically, the constants in the equation for  $\tau_{\rm R}$  have been changed.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely, /S/ Stephen Dembek, Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation Enclosures: Amendment No. 47 to 1. License No. NPF-57 2. Safety Evaluation FAE CENTER CO cc w/enclosures: See next page DISTRIBUTION w/enclosures: RBlough, RGN-I Wanda Jones, 7103 Docket File MO'Brien(2) JCalvo, 11F-22 JWhite, RGN-I NRC & Local PDRs SDembek PDI-2 Reading JStone RJones, 8E23 ACRS(10) OGC SVarga GPA/PA JCalvo DHagan, 3206 OC/LFMB WButler GHill(8) OFC :PDI-2/PM :PDI-2/D :SDembek:rb :CMiller 'tw :RJones NAME : MO В ér 125 /91 :/2//6/91 DATE :////8/91 OFFICIAL RECORD COPY HC AM M81928 **Document Name:** 9112300270 911217 ADOCK 05000354 PDR PDR



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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Stephen Dembek, Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures: 1. Amendment No. 47 to License No. NPF-57 2. Safety Evaluation

cc w/enclosures: See next page Mr. Steven E. Miltenberger Public Service Electric & Gas Co.

#### cc:

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Mr. S. LaBruna Vice President - Nuclear Operations Nuclear Department P.O. Box 236 Hancocks Bridge, New Jersey 08038

Mr. J. J. Hagan General Manager - Hope Creek Operations Hope Creek Generating Station P.O. Box 236 Hancocks Bridge, New Jersey 08038

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Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406 Dr. Jill Lipoti, Asst. Director Radiation Protection Programs NJ Department of Environmental Protection CN 415

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Mr. Scott B. Ungerer, Manager Joint Generation Projects Department Atlantic Electric Company Post Office Box 1500 1199 Black Horse Pike Pleasantville, New Jersey 08232

## Hope Creek Generating Station



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#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### PUBLIC SERVICE ELECTRIC & GAS COMPANY

## ATLANTIC CITY ELECTRIC COMPANY

#### DOCKET NO. 50-354

#### HOPE CREEK GENERATING STATION

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47 License No. NPF-57

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated October 17, 1991 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:
  - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.47, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan. 3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles J. Miller

Charles L. Miller, Director Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 17, 1991

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# ATTACHMENT TO LICENSE AMENDMENT NO. 47

# FACILITY OPERATING LICENSE NO. NPF-57

# DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Overleaf page provided to maintain document completeness.\*

Remove	Insert
3/4 2-3	3/4 2-3
3/4 2-4	3/4 2-4*

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#### POWER DISTRIBUTION LIMITS

## 3/4.2.3 MINIMUM CRITICAL POWER RATIO

#### LIMITING CONDITION FOR OPERATION

3.2.3 The MINIMUM CRITICAL POWER RATIO (MCPR) shall be equal to or greater than the MCPR limit specified in the CORE OPERATING LIMITS REPORT with:

$$\tau = \frac{(\tau_{ave} - \tau_B)}{\tau_A - \tau_B}$$

where:

<sup>t</sup>A = 0.86 seconds, control rod average scram insertion time limit to notch 39 per Specification 3.1.3.3,

$$\tau_{B} = 0.672 + 1.65 \left[ \frac{N_{1}}{n} \right]^{2} (0.016),$$
  
$$\sum_{i=1}^{\Sigma} N_{i}$$

$$\tau_{ave} = \frac{\sum_{i=1}^{n} N_i \tau_i}{\sum_{i=1}^{n} N_i}$$

n = number of surveillance tests performed to date in cycle,

- N<sub>i</sub> = number of active control rods measured in the i<sup>th</sup> surveillance test,
- $\tau_i$  = average scram time to notch 39 of all rods measured in the i<sup>th</sup> surveillance test, and
- $N_1$  = total number of active rods measured in Specification 4.1.3.2.a.

## APPLICABILITY:

OPERATIONAL CONDITION 1, when THERMAL POWER is greater than or equal to 25% of RATED THERMAL POWER.

## ACTION:

a. With the end-of-cycle recirculation pump trip system inoperable per Specification 3.3.4.2, operation may continue and the provisions of Specification 3.0.4 are not applicable provided that, within 1 hour, MCPR is determined to be greater than or equal to the EOC-RPT inoperable limit specified in the CORE OPERATING LIMITS REPORT.

HOPE CREEK

#### POWER DISTRIBUTION LIMITS

## LIMITING CONDITION FOR OPERATION (Continued)

## ACTION: (Continued)

b. With MCPR less than the applicable MCPR limit specified in the CORE OPERATING LIMITS REPORT, initiate corrective action within 15 minutes and restore MCPR to within the required limit within 2 hours or reduce THERMAL POWER to less than 25% of RATED THERMAL POWER within the next 4 hours.

#### SURVEILLANCE REQUIREMENTS

4.2.3 MCPR, with:

- a.  $\tau = 1.0$  prior to performance of the initial scram time measurements for the cycle in accordance with Specification 4.1.3.2, or
- b.  $\tau$  as defined in Specification 3.2.3 used to determine the limit within 72 hours of the conclusion of each scram time surveillance test required by Specification 4.1.3.2,

shall be determined to be equal to or greater than the applicable MCPR limit specified in the CORE OPERATING LIMITS REPORT:

- a. At least once per 24 hours,
- b. Within 12 hours after completion of a THERMAL POWER increase of at least 15% of RATED THERMAL POWER, and
- c. Initially and at least once per 12 hours when the reactor is operating with a LIMITING CONTROL ROD PATTERN for MCPR.
- d. The provisions of Specification 4.0.4 are not applicable.

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

## RELATED TO AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-57

## PUBLIC SERVICE ELECTRIC & GAS COMPANY

## ATLANTIC CITY ELECTRIC COMPANY

#### HOPE CREEK GENERATING STATION

#### DOCKET NO. 50-354

#### 1.0 INTRODUCTION

By letter dated October 17, 1991, the Public Service Electric & Gas Company and Atlantic City Electric Company (the licensees) submitted a request for changes to the Hope Creek Generating Station, Technical Specifications (TS). The requested changes would revise Section 3/4.2.3, MINIMUM CRITICAL POWER RATIO of the TS. Specifically, the constants in the equation for  $\tau_{\rm p}$  would be changed.

#### 2.0 EVALUATION

In a letter, dated April 15, 1991, the General Electric Company informed the licensees that the constants in the equation for  $\tau_B$  had been changed. The changes in the constants were the result of using a new analytical method for the Hope Creek Generating Station (HCGS) reload licensing calculation. The revised constants applied to fuel cycles 2, 3, 4 and future fuel cycles. HCGS is currently operating in fuel cycle 4. The constants are part of an equation defined in TS 3.2.3 which is used to determine  $\tau$ , which in turn is utilized in the CORE OPERATING LIMITS REPORT (COLR) to determine the adjusted operating limit MINIMUM CRITICAL POWER RATIO (MCPR). The  $\tau$  value increases the operating limit MCPR if the control rod insertion times increase appreciably. The actual MCPR is periodically determined in accordance with TS Surveillance Requirement 4.2.3. The actual MCPR must be equal to or greater than the operating limit MCPR.

The licensees stated that the revised constants in the equation for  $\tau_B$  will result in a more conservative value for the operating limit MCPR. The staff has reviewed the licensees' amendment proposal and based on our review, the staff agrees that the substitution of the revised constants in the equation for  $\tau_B$  will result in a more conservative value for  $\tau$  and therefore, the licensees' amendment proposal is acceptable.

Additionally, the licensees stated that the revised constants in the  $\tau_{\rm B}$  equation applied to fuel cycles 2, 3 and 4. Therefore, the licensees were using incorrect and non-conservative values for  $\tau_{\rm B}$  during the above mentioned fuel cycles. The licensees stated that although the values for the  $\tau_{\rm B}$  constants were incorrect, the operating limit MCPRs for the previous cycles were not affected by the revised constants. The licensees stated that the

9112300277 911217 PDR ADUCK 05000354 PDR PDR calculated values ifor  $\tau$  were found to have always been negative for cycles 2, 3 and 4. These calculations were performed by the licensees using both the original and revised constants in the  $\tau_{\rm B}$  equation. The staff reviewed the COLR and verified the licensees' assertion that for any value of  $\tau$  less than zero (i.e. a negative value), the operating limit MCPR is taken as a minimum value. Therefore, only positive values of  $\tau$  can cause the MCPR to be increased. This increase in operating limit MCPR is a penalty incurred for appreciably slower control rod insertion times. Since the HCGS  $\tau$  values were positive throughout cycles 2, 3 and 4, there was no increase in the operating limit MCPR caused by the revised constants in the  $\tau_{\rm R}$  equation.

Therefore, the staff agrees that the licensees did not violate TS 3.2.3 during cycles 2, 3 and 4. This is based on the staff's review of the COLR and on the licensees' statement that there were only negative values of  $\tau$  during cycles 2, 3 and 4.

#### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 57703). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Stephen Dembek

Date: December 17, 1991

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