

March 9, 1992

Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: ONE-TIME EXTENSION OF CONTAINMENT INTEGRATED LEAKAGE RATE TEST
INTERVAL (ILRT), HOPE CREEK GENERATING STATION (TAC NO. M82237)

The Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 29, 1991, and supplemented on January 31, 1992. The supplemental letter did not affect the original no significant hazards determination.

This amendment revised TS 4.6.1.2.a to allow a one-time interval of 56 months between the first and second Type A ILRT. Additionally, the note pertaining to TS 4.6.1.2.d has been deleted. This note allowed a one-time extension, to the first refueling outage, for the Type C test interval, for specific valves.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
/s/

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 48 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:

See next page

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Docket File	SVarga	EJordan, 3701	RBlough, RGN-I
NRC & Local PDRs	JCalvo	GHill(4), P1-37	JWhite, RGN-I
PDI-2 Reading	CMiller	ACRS(10)	CMcCracken, 8D-1
TMurley/FMiraglia	MO'Brien(2)	GPA/PA	OGC
JPartlow	SDembek	OC/LFMB	RLobel, 17G-21
CRossi			

OFFICE	PDI-2/A	PDI-2/PM	PDI-2/D	SPLB	OGC
NAME	MO'Brien	SDembek:rb	CMiller	CMcCracken	
DATE	2/24/92	2/12/91	3/19/91	2/13/91	2/18/91

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, appearing to read "Stephen Dembek".

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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1. Amendment No. 48 to
License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas
Company

Hope Creek Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated November 29, 1991, and supplemented on January 31, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 48, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 9, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 48

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-3
3/4 6-4

Insert

3/4 6-3
3/4 6-4

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, valves which form the boundary for the long-term seal of the feedwater lines, and other valves which are hydrostatically tested per Table 3.6.3-1, subject to Type B and C tests to less than or equal to $0.60 L_a$, and
- c. The leakage rate to less than or equal to 46.0 scfh combined through all four main steam lines, and
- d. The combined leakage rate for all containment isolation valves which form the boundary for the long-term seal of the feedwater lines in Table 3.6.3-1 to less than or equal to 10 gpm, and
- e. The combined leakage rate for all other penetrations and containment isolation valves in hydrostatically tested lines in Table 3.6.3-1 which penetrate the primary containment to less than or equal to 10 gpm.

prior to increasing reactor coolant system temperature above 200°F.

SURVEILLANCE REQUIREMENTS.

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals** during shutdown at P_a , 48.1 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
- b. If any periodic Type A test fails to meet $0.75 L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $0.75 L_a$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $0.75 L_a$, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_a$.

*Exemption to Appendix "J" of 10 CFR 50.

**The maximum permissible test interval for the second Type A test of the first ten year service period is extended to 56 months. This extension expires upon completion of the second Type A test of the first ten year service period.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test, to be between $0.75 L_a$ and $1.25 L_a$.

The formula to be used is: $[L_o + L_{am} - 0.25 L_a] \leq L_c \leq [L_o + L_{am} + 0.25 L_a]$ where $L_c \equiv$ supplement test result; $L_o \equiv$ superimposed leakage; and $L_a \equiv$ measured Type A leakage.

- d. Type B and C tests shall be conducted with gas at P_a , 48.1 psig*, at intervals no greater than 24 months except for tests involving:
 1. Air locks,
 2. Main steam line isolation valves,
 3. Valves pressurized with fluid from a seal system,
 4. All containment isolation valves in hydrostatically tested lines in Table 3.6.3-1 which penetrate the primary containment, and
 5. Purge supply and exhaust isolation valves with resilient material seals.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. Containment isolation valves which form the boundary for the long-term seal of the feedwater lines in Table 3.6.3-1 shall be hydrostatically tested at $1.10 P_a$, 52.9 psig, at least once per 18 months.
- h. All containment isolation valves in hydrostatically tested lines in Table 3.6.3-1 which penetrate the primary containment shall be leak tested at least once per 18 months.
- i. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Surveillance Requirements 4.6.1.8.2.
- j. The provisions of Specification 4.0.2 are not applicable to Specifications 4.6.1.2.a, 4.6.1.2.b, 4.6.1.2.c, 4.6.1.2.d, and 4.6.1.2.e.

*Unless a hydrostatic test is required per Table 3.6.3-1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated November 29, 1991, and supplemented on January 31, 1992, the Public Service Electric & Gas Company (PSE&G) and Atlantic City Electric Company (the licensees) submitted a request for changes to the Hope Creek Generating Station (HCGS) Technical Specifications (TS). The requested changes would revise TS 4.6.1.2.a to allow a one-time interval of 56 months between the first and second Type A Containment Integrated Leakage Rate Tests (ILRT). The present TS interval is 40 ± 10 months. Additionally, the note pertaining to TS 4.6.1.2.d would be deleted. This note allowed a one-time extension, to the first refueling outage, for the Type C test interval, for specific valves. This note is no longer applicable. The supplemental letter did not affect the original no significant hazards determination.

2.0 EVALUATION

Paragraph III.D.1(a) of 10 CFR Part 50, Appendix J (Appendix J) states: "After the preoperational leakage rate tests, a set of three Type A tests shall be performed, at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections." In addition to the above quoted Appendix J requirement, Section 4.6.1.2.a of the HCGS TS also specifies that: "Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P_a , 48.1 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection."

In its November 29, 1991 submittal, the licensee stated that the first operational Type A ILRT of the Hope Creek containment was successfully completed during the second refueling outage in November 1989. This was about 33 months after the plant commenced commercial operations. The third Type A ILRT must be completed during the 10-year inservice inspection (ISI) outage scheduled for March through May 1997. Therefore, an option exists as to whether to perform the second test during the fourth or fifth refueling

outage. Completion of the second ILRT during the fourth refueling outage, scheduled for September through November 1992, would result in an interval of 34-36 months between the first and second tests, thus fulfilling the TS limit of 40 ± 10 months. However, this would then result in an interval of 54-56 months between the second and third tests, which exceeds the TS limit by 4 to 6 months. Completion of the second ILRT during the fifth refueling outage, scheduled for March through April 1994, would result in intervals of 52-53 months between the first and second tests and 35-38 months between the second and third tests. In this case the second test interval would exceed the TS limit by 2 or 3 months. The licensee provided the following justification to support its proposed amendment:

1. The intent of the established testing intervals is to conduct three tests at approximately equal intervals within a given ten year period with the third test coinciding with the 10-year ISI outage.
2. The first Type A test was completed successfully during the second refueling outage in November 1989. Since then, there has not been any modifications made to the plant which could adversely effect the test results.
3. Type B and C tests have been completed satisfactorily during each of the three previous Hope Creek outages and are scheduled to be performed during the upcoming fourth refueling outage. Demonstrated operability of the associated components and penetrations provides additional assurance that the integrated containment leakage rate remains satisfactory.
4. Additionally, it should be noted that the most recent Proposed Rule, Appendix J to 10 CFR Part 50 (April 8, 1991), Section III.A.3.c allows a single Type A test interval to be extended for up to 25% of the specified 4-year interval (i.e. up to 60 months). The proposed change to TS 4.6.1.2.a to the fifth refueling outage would remain within the test frequency allowed in the proposed rule.
5. This is a one-time Type A test interval extension request. A new Type A test schedule will be preplanned for the next 10-year service period.

In its January 31, 1992 letter, PSE&G revised its initial request. The TS changes that were originally proposed required the second Type A test to be conducted during the fifth refueling outage. In order to provide the flexibility of performing the test prior to the fifth refueling outage due to unforeseen contingencies, PSE&G proposed a maximum 56-month limit between the first and second tests. PSE&G is still planning to conduct the Type A test during the fifth refueling outage. The request for a 56-month interval is based on 53 months between the second and fifth refueling outages plus 3 months for unforeseen scheduling contingencies.

The staff has reviewed PSE&G's submittals and finds that TS 4.6.1.2.a was written to explicitly quantify the Appendix J statement: "... a set of three Type A tests shall be performed at approximately equal intervals during each 10 year service period." The intent of the Appendix J statement and TS 4.6.1.2.a was to allow the licensee to perform the ILRTs at the refueling shutdowns that would provide approximately equal spacing between the ILRTs. It was not intended for the licensee to conduct a shortened cycle in order to perform the ILRT within the 40 ± 10 month TS 4.6.1.2.a surveillance interval. Since the licensee must conduct the third ILRT during the scheduled shutdown for the 10-year ISI and because PSE&G has justified the leaktight integrity of the containment based on previous leakage test results, the staff concludes that a one-time delay of 6 months beyond the permitted test interval will not, for the reasons stated above, have a significant safety impact. The staff, therefore, concludes that PSE&G's proposed TS amendment requesting a delay in conducting the second ILRT meets the intent of Appendix J and is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 66928). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: March 9, 1992