

April 1, 1992

- Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: TECHNICAL SPECIFICATION 3.11.2.6 AMENDMENT, HOPE CREEK GENERATING
STATION (TAC NO. M81929)

The Commission has issued the enclosed Amendment No. 49 to Facility Operating
License No. NPF-57 for the Hope Creek Generating Station. This amendment
consists of changes to the Technical Specifications (TSs) in response to your
application dated October 17, 1991.

This amendment revised the Explosive Gas Mixture and the Radioactive Gaseous
and Liquid Effluent Monitoring Instrumentation section in the TSs.
Specifically, TS 3.11.2.6, ACTION b, has been revised to agree with the
corresponding ACTION b of TS 3.3.7.11 and ACTION 124 of Table 3.3.7.11-1.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 49 to License No. NPF-57
- 2. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Stephen Dembek".

Stephen Dembek, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas
Company

Hope Creek Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated October 17, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 49, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles L. Miller

Charles L. Miller, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 49

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Overleaf page provided to maintain document completeness.*

Remove

3/4 11-15
3/4 11-16

Insert

3/4 11-15*
3/4 11-16

RADIOACTIVE EFFLUENTS

VENTILATION EXHAUST TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.11.2.5 The VENTILATION EXHAUST TREATMENT SYSTEM for the Reactor Building and the Service and Radwaste Building shall be OPERABLE and appropriate portions of this system shall be used to reduce release of radioactivity when the projected doses in 31 days due to gaseous effluent releases from each unit to areas at and beyond the SITE BOUNDARY (see Figure 5.1.1-1) would exceed:

- a. 0.2 mrad to air from gamma radiation, or
- b. 0.4 mrad to air from beta radiation, or
- c. 0.3 mrem to any organ of a MEMBER OF THE PUBLIC

APPLICABILITY: At all times.

ACTION:

- a. With radioactive ventilation exhaust being discharged without treatment and in excess of the above limits, prepare and submit to the Commission within 30 days pursuant to Specification 6.9.2 a Special Report that includes the following information:
 1. Identification of any inoperable equipment or subsystems, and the reason for the inoperability,
 2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and
 3. Summary description of action(s) taken to prevent a recurrence.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.5.1 Doses due to gaseous releases from each unit to areas at and beyond the SITE BOUNDARY shall be projected at least once per 31 days in accordance with the methodology and parameters in the ODCM, when the VENTILATION EXHAUST TREATMENT SYSTEM is not being fully utilized.

4.11.2.5.2 The installed VENTILATION EXHAUST TREATMENT SYSTEM shall be considered OPERABLE by meeting Specifications 3.11.2.1 and 3.11.2.2 and 3.11.2.3.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

3.11.2.6 The concentration of hydrogen in the main condenser offgas treatment system shall be limited to less than or equal to 4% by volume.

APPLICABILITY: At all times.

ACTION:

- a. With the concentration of hydrogen in the main condenser offgas treatment system exceeding the limit, restore the concentration to within the limit within 48 hours.
- b. With continuous monitors inoperable, take the actions required by Specification 3.3.7.11, ACTION b.
- c. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.2.6 The concentration of hydrogen in the main condenser offgas treatment system shall be determined to be within the above limits by continuously monitoring the waste gases in the main condenser offgas treatment system whenever the main condenser evacuation system is in operation with the hydrogen monitors required OPERABLE by Table 3.3.7.11-1 of Specification 3.3.7.11.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated October 17, 1991, the Public Service Electric & Gas Company and Atlantic City Electric Company (the licensees) submitted a request for changes to the Hope Creek Generating Station (HCGS), Technical Specifications (TS). The requested changes would revise the Explosive Gas Mixture and the Radioactive Gaseous and Liquid Effluent Monitoring Instrumentation section in the TS. Specifically, TS 3.11.2.6, ACTION b would be revised to agree with the corresponding ACTION b of TS 3.3.7.11 and ACTION 124 of Table 3.3.7.11-1.

2.0 EVALUATION

The TS for Facility Operating License No. NPF-57 (FOL) for the HCGS were issued in July 1986. The Radiological Effluent Technical Specifications (RETS) were included as part of the Hope Creek TS and were implemented by the licensee upon issuance of the TS. By letter dated September 12, 1986, and supplemented by letters dated September 22, 1986, and November 10, 1986, the licensees requested changes to the RETS incorporated in the TS. The changes were requested to modify TS 3.3.7.9 and TS 3.3.7.10 to be consistent with Revision 2 to the Standard RETS. The changes were approved by the U.S. Nuclear Regulatory Commission Staff (the Staff) in Amendment 2 to the FOL. Subsequently, Amendment 10 to the FOL renumbered TS 3.3.7.9 and TS 3.3.7.10 as 3.3.7.10 and 3.3.7.11, respectively. The content of TS 3.3.7.10 and TS 3.3.7.11 (formerly, TS 3.3.7.9 and TS 3.3.7.10) was not changed by Amendment 10 to the FOL.

The licensees inadvertently omitted TS 3.11.2.6 when they submitted the license amendment request that led to Amendment 2 to the FOL. Therefore, there are currently two HCGS TS that delineate actions required in the event that the main condenser offgas treatment system hydrogen monitors are declared inoperable. The two TS for hydrogen monitors differ slightly in their ACTION statement wording. TS 3.11.2.6, ACTION b states: "With continuous monitors inoperable, operation of the main condenser offgas treatment system may continue for up to 30 days provided grab samples are collected at least once per 4 hours and analyzed within the following 4 hours." As written, this TS would require the licensees to secure discharging through the main condenser offgas treatment system after 30 days even if they are performing grab samples every four hours. This is contrary to the staffs' guidance delineated in NUREG-0473 (Revision 2). TS 3.3.7.11, ACTION b, in conjunction with ACTION 124 of TS TABLE 3.3.7.11-1, also delineates actions to be taken for inoperable

main condenser offgas treatment system hydrogen monitors. TS 3.3.7.11, ACTION b in conjunction with ACTION 124 of TS Table 3.3.7.11-1, contains the same grab sample requirement as does TS 3.11.2.6, ACTION b; however, it does not restrict the discharge to 30 days. Instead of the 30-day restriction, TS 3.3.7.11 states in part: "Exert best efforts to return the instruments to OPERABLE status within 30 days and, if unsuccessful, explain in the next Semiannual Radioactive Effluent Release Report pursuant to Specification 6.9.1.7 why this inoperability was not corrected in a timely manner." This wording in TS 3.3.7.11 was modified by Amendment 2 to the FOL based on the guidance in NUREG-0473 (Revision 2).

The license amendment requested on October 17, 1991, would make the ACTION statement for TS 3.11.2.6, ACTION b agree with the applicable ACTION statements of TS 3.3.7.11. This change will eliminate conflicting requirements for the hydrogen monitoring instrumentation. The licensees' proposed TS amendment is consistent with the intent of the Radiological Effluent Technical Specifications (RETS), namely, that alternative radioactive effluent monitoring techniques are used to assess the effluents should the primary means not be available. The HCGS TS include the hydrogen monitoring instrumentation as part of the Radioactive Effluent monitoring instrumentation.

Based on the staff's review of the licensees' October 17, 1991, submittal and HCGS FOL Amendments 2 and 10, the staff finds that the licensees' proposed TS amendment request corrects an unintentional omission of TS 3.11.2.6, ACTION b when Amendment 2 to the FOL was issued and meets the intent of NUREG-0473 (Revisions 2 and 3). Therefore, the licensees' proposal is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 57702). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: April 1, 1992