July 7, 1987

Docket No. 50-354

Mr. Corbin A. McNeill, Jr.
Senior Vice President - Nuclear
Public Service Electric & Gas Company
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. McNeill:

SUBJECT: SECONDARY CONTAINMENT DAMPER CLOSURE TIME (TAC NO. 64950)

Re: HOPE CREEK GENERATING STATION

The Commission has issued the enclosed Amendment No. 6 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 13, 1987, as supplemented March 26, 1987.

This amendment reduces the maximum isolation times for the secondary containment ventilation automatic isolation dampers.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

George Rivenbark

George Rivenbark, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 6 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

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NRC PDR	OGC - Bethesda	EButcher	RGallo
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PDI-2 Reading	EJordan	ACRS (10)	
WButler	JPartlow	CMiles, GPA/PA	
GRivenbark/RLo	TBarnhart (4)	RDiggs, ARM/LFMB	

Previously concurred*:

LA:PDI-2:DRPI/II*	PM:PDI-2:DRPI/II*	OGC*	D:PDI-2:DRPI/II
MO'Brien	GRivenbark:ca	EChan	WButler
06/18/87	06/18/87	06/26/87	07/7/87

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated March 13, 1987, as supplemented March 26, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 6 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1987

LA:PDI-2:DRPI/II	PM:PDI-2:DRPI/II	OGC	D:PDI-2:DRPI/II
MO'Brien	GRivenbark	E.Chan	WButler
6/18/87	6/18/87	6/26/87	7/7/87

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf page provided to maintain document completeness.*

Remove	Insert
3/4 6-49	3/4 6-49*
3/4 6-50	3/4 6-50

SECONDARY CONTAINMENT AUTOMATIC ISOLATION DAMPERS

LIMITING CONDITION FOR OPERATION

3.6.5.2 The secondary containment ventilation system (RBVS) automatic isolation dampers shown in Table 3.6.5.2-1 shall be OPERABLE with isolation times less than or equal to the times shown in Table 3.6.5.2-1.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3 and *.

ACTION:

With one or more of the secondary containment ventilation system automatic isolation dampers shown in Table 3.6.5.2-1 inoperable, maintain at least one isolation damper OPERABLE in each affected penetration that is open and within 8 hours either:

a. Restore the inoperable dampers to OPERABLE status, or

b. Isolate each affected penetration by use of at least one deactivated damper secured in the isolation position, or

c. Isolate each affected penetration by use of at least one closed manual valve or blind flange.

Otherwise, in OPERATIONAL CONDITION 1, 2 or 3, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

Otherwise, in Operational Condition *, suspend handling of irradiated fuel in the secondary containment, CORE ALTERATIONS and operations with a potential for draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.6.5.2 Each secondary containment ventilation system automatic isolation damper shown in Table 3.6.5.2-1 shall be demonstrated OPERABLE:

a. Prior to returning the damper to service after maintenance, repair or replacement work is performed on the damper or its associated actuator, control or power circuit by cycling the damper through at least one complete cycle of full travel and verifying the specified isolation time.

b. During COLD SHUTDOWN or REFUELING at least once per 18 months by verifying that on a containment isolation test signal each isolation damper actuates to its isolation position.

c. By verifying the isolation time to be within its limit at least once per 92 days.

*When irradiated fuel is being handled in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

TABLE 3.6.5.2-1

SECONDARY CONTAINMENT VENTILATION SYSTEM AUTOMATIC ISOLATION DAMPERS

ISOLATION GROUP NO. 19

DAMPER FUNCTION	MAXIMUM ISOLATION TIME (Seconds)
1. Reactor Building Ventilation Supply Damper HD-9370A	7
2. Reactor Building Ventilation Supply Damper HD-9370B	7
3. Reactor Building Ventilation Exhaust Damper HD-9414A	7
4. Reactor Building Ventilation Exhaust Damper HD-9414B	7

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated March 13, 1987, as supplemented March 26, 1987, Public Service Electric & Gas Company requested an amendment to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. The proposed amendment would reduce the maximum isolation times for the four secondary containment ventilation system automatic isolation dampers listed in Technical Specification (TS) Table 3.6.5.2-1 from 10 seconds to 7 seconds.

2.0 EVALUATION

The licensee states that the change is required to conservatively correct the TSs and satisfy the functional design basis for the system as stated in the Final Safety Analysis Report (FSAR). FSAR Section 9.4.2.3 states that "the exhaust air transit time between the refueling area monitors and the Reactor Building Ventilation System (RBVS) exhaust system isolation dampers is greater than the combined time of damper closure and the monitor response. The exhaust air transit time is 12 seconds, and the combined monitor response and valve closing time is within 11 seconds." Current TSs allow a monitor response time of $\leq$ 4.0 seconds (TS Table 3.3.2-3) and a damper closure time of $\leq$ 10 seconds (TS Table 3.6.5.2-1) which yields a combined monitor response/damper closure time of $\leq$ 14 seconds. Since this value is in conflict with, and less conservative than, the functional design requirements, the licensee proposes revising the damper isolation time from $\leq$ 10 seconds to $\leq$ 7 seconds in TS Table 3.6.5.2-1 to achieve the original design intent.

The staff review of the proposed change determined that the secondary containment is designed to minimize any ground level release of radioactive material which may result from a postulated accident. The NRC's Standard Review Plan, NUREG-0800, (Section 9.4.2) requires that the system have the capability to detect and control leakage of radioactive contamination from the system and include the capability of automatically isolating the normal ventilation system before the first contaminated airborne particles and gases reach the normal ventilation

exhaust ducts. The proposed change would ensure that this capability exists by requiring that the maximum combined monitor response/damper closure time be less than the exhaust air transit time. The staff, therefore, concludes that the proposed change in secondary containment ventilation system automatic isolation dampers maximum isolation time is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 13348) on April 22, 1987, and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: L. J. Wink

Dated: July 7, 1987

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		6/24/87	