May 15, 1989

Docket No. 50-354

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger

SUBJECT: INCREASE, ON A ONE TIME BASIS, THE 18 MONTH SURVEILLANCE INTERVAL BY APPROXIMATELY 2 MONTHS FOR THE A AND D EMERGENCY DIESEL GENERATORS (TAC NO. 71984)

Re: HOPE CREEK GENERATING STATION

The Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 2, 1989.

This amendment increases, on a one time basis, the 18 month surveillance interval by approximately 2 months for the A and D Emergency Diesel Generators (EDGs).

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Clyde Shiraki, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 25 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

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CShiraki(3)/SBrown	TMeek (4)	RDiggs, ARM/LFMB	SNSaba

[MILTE LETTER]

PDI-2/LA	PDI-2/PM	PDI-2/D	OGC
MO'Brien	CShiraki:tc	WButler	
5/12/89	5/12/89	5/15/89	5/18/89

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Mr. Steven E. Miltenberger
Public Service Electric & Gas Co.

Hope Creek Generating Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated February 2, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 25 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 15, 1989

PDI-2/LA MO'Brien 4/27/89	PDI-2/PM CShinaki:tr 4/4/89	PDI-2/D WButler 5/5/89	OGC 5/3/89

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 15, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 25

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Overleaf page is provided to maintain document completeness.*

Remove	Insert
3/4 8-5*	3/4 8-5*
3/4 8-6	3/4 8-6

SURVEILLANCE REQUIREMENTS (Continued)

6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.

7. Verifying the pressure in all diesel generator air start receivers to be greater than or equal to 325 psig.

8. Verifying the lube oil pressure, temperature and differential pressure across the lube oil filters to be within manufacturer's specifications.

b. At least once per 31 days by visually examining a sample of lube oil from the diesel engine to verify absence of water.

c. At least once per 31 days and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the fuel oil day tank.

d. At least once per 92 days by removing accumulated water from the fuel oil storage tanks.

e. At least once per 31 days by performing a functional test on the emergency load sequencer to verify operability.

f. At least once per 92 days and from new fuel oil prior to addition to the storage tanks by obtaining a sample in accordance with ASTM-D4057 and by verifying that the sample meets the following minimum requirements and is tested within the specified time limits:

1. As soon as sample is taken or from new fuel prior to addition to the storage tank, as applicable, verify in accordance with the tests specified in ASTM-D975-77 that the sample has:

a) A water and sediment content of less than or equal to 0.05 volume percent.

b) A kinematic viscosity @ 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes or a Saybolt Second Universal (SSU) viscosity at 100°F of greater than or equal to 32 SSU but less than or equal to 40.1 SSU.

c) A specific gravity as specified by the manufacturer as API gravity @ 60°F of greater than or equal to 28 degrees but less than or equal to 42 degrees.

2. Within one week after obtaining the sample, verify an impurity level of less than 2 mg of insolubles per 100 ml. when tested in accordance with ASTM-D2274-70.

SURVEILLANCE REQUIREMENTS (Continued)

3. Within 2 weeks after obtaining the sample, verify that the other properties specified in Table 1 of ASTM-D975-77 and Regulatory Guide 1.137, Position 2.a, are met when tested in accordance with ASTM-D975-77.

g. At least once per 2 months, by verifying the buried fuel oil transfer piping's cathodic protection system is OPERABLE and at least once per year by subjecting the cathodic protection system to a performance test.

h. At least once per 18 months #,## during shutdown, by:

1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.

2. Verifying the diesel generator capability to reject a load of greater than or equal to that of the RHR pump motor (1003 kW) for each diesel generator while maintaining voltage at 4160 ± 420 volts and frequency at 60 ± 1.2 Hz.

3. Verifying the diesel generator capability to reject a load of 4430 kW without tripping. The generator voltage shall not exceed 4580 volts during and following the load rejection.

4. Simulating a loss of offsite power by itself, and:

a) Verifying loss of power is detected and deenergization of the emergency busses and load shedding from the emergency busses.

b) Verifying the diesel generator starts* on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds after receipt of the start signal, energizes the autoconnected shutdown loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the shutdown loads. After energization, the steady state voltage and frequency of the emergency busses shall be maintained at 4160 ± 420 volts and 60 ± 1.2 Hz during this test.

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*This diesel generator start (10 sec) and subsequent loading (130 sec) from ambient conditions may be preceded by an engine prelube period and/or other warmup procedures recommended by the manufacturer so that mechanical stress and wear on the diesel engine is minimized.

#For any start of a diesel generator, the diesel must be loaded in accordance with the manufacturer's recommendations.

##The surveillance interval for Emergency Diesel Generators 1AG400 and 1DG400, overdue beginning August 19 and 27, 1989 respectively, is extended until October 15, 1989 and the provisions of Specification 4.0.2.a are not applicable for this one-time extension (i.e., this surveillance interval will be considered as an 18-month interval). With regard to the provisions of Specification 4.0.2.b, the combined time interval accrued for previous consecutive intervals for this surveillance section by all four Emergency Diesel Generators, on a one-time basis, is considered to be zero upon completion, during the second refueling outage, of all 18-month surveillance of this Specification.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO.25 TO FACILITY OPERATING LICENSE NO. NPF-57

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

1.0 INTRODUCTION

By letter dated February 2, 1989, Public Service Electric & Gas Company (PSE&G) requested an amendment to Facility Operating License No. NPF-57 for the Hope Creek Generating Station (HCGS). The proposed amendment would revise, by addition of a footnote, Technical Specification Surveillance Requirement 4.8.1.1.2h to increase the 18 month plus 25% surveillance interval for the A and D Emergency Diesel Generators (EDGs), overdue on August 19 and 27, 1989 respectively, until October 15, 1989.

2.0 EVALUATION

HCGS has four Emergency Diesel Generators (EDG) and the PSE&G surveillance requirements of these EDGs is that the units should be subjected to 18 month inspection and verification of the EDG capabilities in accordance with established procedures. The Surveillance Requirement shall be performed within an 18 month interval with a maximum allowable extension not to exceed 25% of the surveillance interval.

The proposed change to HCGS Technical Specifications (TS) is to permit a one-time extension of the 18 month plus 25% surveillance interval required by TS 4.8.1.1.2h, by approximately two months until not later than October 15, 1989. This amendment will be accomplished by adding a footnote to T.S. 4.8.1.1.2h extending the surveillance interval to October 15, 1989.

PSE&G's original plan to conduct an 18 month plus 25% EDG surveillance is on a staggered basis (half at refueling outages and half during mid-cycle outages). Based on this program the A and D EDGs are scheduled for their 18 month overhaul inspection during the coming Cycle 2 (mid-cycle) outage from February 18 through March 10, 1989 or based on the 25% extension the overhaul becomes overdue on August 19 and 27, 1989 respectively. However, due to a PSE&G decision, no future mid-cycle outages are planned during the 18 month fuel cycles. The next refueling outage is scheduled

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for September 23, through November 6, 1989. The same overhaul inspection must again be performed on EDG A and D so that the 18 month surveillance interval for all four EDGs will coincide with future refueling outages.

EDGs A and D were overhauled on September 22 and 30, 1987 respectively during Cycle 1 mid-cycle outage with approximate accumulated operational hours of 97 hours for EDG A and 102 hours for EDG D. The results of the overhauls were satisfactory and no abnormal conditions were present. Since the overhaul and through December 8, 1988, EDG A has accumulated 19 operational hours and EDG D has accumulated 32 operational hours. It is expected that an additional 18 operational hours per each EDG will be accumulated through late August 1989 and another additional 4 hours until October 15, 1989. PSE&G's requested extension of the EDG surveillance represents only an additional 4 hours of operation on each unit based on 2 hours per month usage. This amount of runtime during the period is small and the wear failure has a very low probability. The delaying of the overhaul would not affect the performance and reliability of the EDG.

Since each inspection/overhaul refurbishment of the EDGs established the engines within the manufacturer's specifications or restored them to these specifications, the EDGs are essentially "as good as new" after the overhaul. The reinitialization of the 18 month interval to zero will have a minimal impact on their reliability or operability.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (54 FR 11841) on March 22, 1989 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributors: C. Y. Shiraki, S. N. Saba

Dated: May 15, 1989