

Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385



JUN 14 2001

Docket No. 50-336
B18423

RE: 10 CFR 50.90

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 2
Changes to Technical Specifications
Updating List of Documents Describing the Analytical Methods
Specified in Technical Specification 6.9.1.8b (TSCR 2-4-01)
Revised Response to Questions 1 and 3 in the
Significant Hazards Considerations

In a letter dated April 11, 2001,⁽¹⁾ Dominion Nuclear Connecticut, Inc. (DNC) submitted a license amendment associated with the list of documents describing the analytical methods specified in Technical Specification 6.9.1.8b.

In a conference call with the Nuclear Regulatory Commission (NRC) conducted on June 4, 2001, it was identified that the response to Questions 1 and 3 of the Significant Hazards Consideration (SHC) contained in the letter dated April 11, 2001,⁽¹⁾ did not support the conclusion that the proposed changes will not increase the probability or consequences of an accident previously evaluated and will not result in a reduction in a margin of safety.

DNC has revised the responses to Questions 1 and 3 of the SHC (Attachment 1). The revised responses will not affect the conclusion of the Safety Summary and the SHC contained in DNC's letter dated April 11, 2001.⁽¹⁾

Additionally, there are no other changes to any other aspect of the previous submittal.

⁽¹⁾ R. P. Necci letter to the NRC, "Millstone Nuclear Power Station, Unit No. 2, Changes to Technical Specifications, Updating List of Documents Describing the Analytical Methods Specified in Technical Specification 6.9.1.8b (TSCR 2-4-01)," dated April 11, 2001.

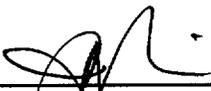
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There are no regulatory commitments contained within this letter.

If you should have any questions on the above, please contact Mr. Ravi Joshi at 860-440-2080.

Very truly yours,

DOMINION NUCLEAR CONNECTICUT, INC.



J. Alan Price, Vice President
Nuclear Technical Services - Millstone

Sworn to and subscribed before me

this 14 day of June, 2001

Donna Lynne Williams
Notary Public

My Commission expires Nov 30, 2001

Attachment (1)

cc: H. J. Miller, Region I Administrator
J. I. Zimmerman, NRC Project Manager, Millstone Unit No. 2
S. R. Jones, Senior Resident Inspector, Millstone Unit No. 2

Director
Bureau of Air Management
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm Street
Hartford, CT 06106-5127

Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Changes to Technical Specifications
Updating List of Documents Describing the Analytical Methods
Specified in Technical Specification 6.9.1.8b (TSCR 2-4-01)
Revised Response to Questions 1 and 3 in the
Significance Hazards Consideration

**Proposed Revision to Technical Specifications
Updating List of Documents Describing the Analytical Methods Specified
in Technical Specification 6.9.1.8b (TSCR 2-4-01)
Significant Hazards Consideration**

Significant Hazards Consideration

In accordance with 10 CFR 50.92, Dominion Nuclear Connecticut, Inc. (DNC) has reviewed the proposed changes and has concluded that they do not involve a Significant Hazards Consideration (SHC). The basis for this conclusion is that the three criteria of 10 CFR 50.92(c) are not compromised. The proposed changes do not involve an SHC because the changes would not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change in document 6 and the deletion of document 7 of Technical Specification 6.9.1.8b are made to identify the most recent, Nuclear Regulatory Commission (NRC) approved, model used in Small Break Loss of Coolant Accident (SBLOCA) applications. This methodology meets the requirements of 10 CFR 50.46 and 10 CFR 50 Appendix K. This change has no impact on plant equipment operation. Since the change only affects the SBLOCA analysis, it cannot affect the likelihood or consequences of accidents. Therefore, this change will not increase the probability or consequences of an accident previously evaluated.

The proposed change in document 15 (renumbered 14) of Technical Specification 6.9.1.8b is made to identify the most recent, NRC approved, setpoint methodology for Combustion Engineering type reactors. This change has no impact on plant equipment operation. The proposed change does not revise any setpoints assumed in the accident analyses. Therefore, it cannot affect the likelihood or consequences of accidents. Therefore, this change will not increase the probability or consequences of an accident previously evaluated.

The proposed change to add a new document as 6.9.1.8b.15 is required to identify the most recent Non-LOCA methodology to be used in the Millstone Unit No. 2 Non-LOCA analysis. The use of this methodology will demonstrate that the acceptance criteria for Non-LOCA events are met. This change has no impact on plant equipment operation. The change does not affect the acceptance criteria for Non-LOCA accident. Therefore, it cannot affect the likelihood or consequences of accidents. Therefore, this change will not increase the probability or consequences of an accident previously evaluated.

Deleting the revision number and the date from the documents contained in sections 6.9.1.8b.1 through 6.9.1.8b.15 has no impact on the actual analytical methods used to determine the core operating limits, nor does it have impact on the calculations performed for current or future reloads. This change is administrative in nature. This change has no impact on plant equipment operation nor does it affect the likelihood or consequences of accidents. Therefore, this change will not increase the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes will not alter the plant configuration (no new or different type of equipment will be installed) or require any new or unusual operator actions. They do not alter the way any structure, system, or component functions and do not alter the manner in which the plant is operated. These changes do not introduce any new failure modes. Therefore, the proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed changes have no impact on plant equipment operation. The proposed changes do not revise any setpoints assumed in the analyses and do not affect the acceptance criteria for Non-LOCA accidents. Therefore, the proposed changes will not result in a reduction in a margin of safety.