

November 24, 1987

Docket No. 50-354

Mr. Corbin A. McNeill, Jr.
Senior Vice President - Nuclear
Public Service Electric & Gas Company
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. McNeill:

SUBJECT: SAFETY PARAMETER DISPLAY SYSTEM LICENSE CONDITION (TAC NO. 66043)

Re: HOPE CREEK GENERATING STATION

The Commission has issued the enclosed Amendment No. 13 to Facility Operating License No. NPF-57 for the Hope Creek Generating Station. This amendment consists of a change to License Condition 2.C.(13) in response to your application dated August 18, 1987.

This amendment extends the required schedule for having four additional Safety Parameter Display System parameters operational.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

8712030268 871124
PDR ADDCK 05000354
P PDR

/s/
George Rivenbark, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 13 to License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, reading "George Rivenbark", is written over the typed name.

George Rivenbark, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

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License No. NPF-57
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. C. A. McNeill
Public Service Electric & Gas Co.

Hope Creek Generating Station

cc:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. NPF-57

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated August 18, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, paragraph 2.C.(13) of Facility Operating License No. NPF-57 is hereby amended as indicated on the attached license page.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Change to page 6 of License
No. NPF-57

Date of Issuance: November 24, 1987

LA: PDI-2:DRPI/II
MO'Brien
11/15/87

PM: PDI-2:DRPI/II
GRivenbark
11/14/87

11/18/87
D: PDI-2:DRPI/II
WButler
11/23/87

BC: PDI-2:DLPO
WRegan
11/9/87

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Change to page 6 of License
No. NPF-57

Date of Issuance: November 24, 1987

(13) Safety Parameter Display System (Section 18.2, SSER No. 5)

Prior to the earlier of 90 days after restart from the first refueling outage or July 12, 1988, PSE&G shall add the following parameters to the SPDS and have them operational:

- a. Primary containment radiation
- b. Primary containment isolation status
- c. Combustible gas concentration in primary containment
- d. Source range neutron flux

- D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. An exemption from the criticality alarm requirements of 10 CFR 70.24 was granted in Special Nuclear Material License No. 1953, dated August 21, 1985. This exemption is described in Section 9.1 of Supplement No. 5 to the SER. This previously granted exemption is continued in this operating license. An exemption from certain requirements of Appendix A to 10 CFR Part 50, is described in Supplement No. 5 to the SER. This exemption is a scheduler exemption to the requirements of General Design Criterion 64, permitting delaying functionality of the Turbine Building Circulating Water System-Radiation Monitoring System until 5 percent power for local indication, and until 120 days after fuel load for control room indication (Appendix R of SSER 5). Exemptions from certain requirements of Appendix J to 10 CFR Part 50, are described in Supplement No. 5 to the SER. These include (a) an exemption from the requirement of Paragraph III.D.2(b)(ii) of Appendix J, exempting overall containment air lock leakage testing unless maintenance has been performed on the air lock that could affect air lock sealing capability (Section 6.2.6 of SSER 5); (b) an exemption from the requirement of Paragraph III.C.2(b) of Appendix J, exempting main steam isolation valve leak-rate testing at 1.10 Pa (Section 6.2.6 of SSER 5); (c) an exemption from Paragraph III.D.3 of Appendix J, exempting Type C testing on traversing incore probe system shear valves (Section 6.2.6 of SSER 5); (d) an exemption from Paragraph III.D.2(a) of Appendix J, exempting Type C testing for instrument lines and lines containing excess flow check valves (Section 6.2.6 of SSER 5); and (e) an exemption from Paragraph III.C.2(a) of Appendix J, exempting Type C testing of thermal relief valves (Section 6.2.6 of SSER 5). These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. These exemptions are hereby granted. The special circumstances regarding each exemption are identified in the referenced section of the safety evaluation report and the supplements thereto. These exemptions are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO.13 TO FACILITY OPERATING LICENSE NO. NPF-57
PUBLIC SERVICE ELECTRIC & GAS COMPANY
ATLANTIC CITY ELECTRIC COMPANY
HOPE CREEK GENERATING STATION
DOCKET NO. 50-354

1.0 INTRODUCTION

A License Condition 2.C.(13) to the Hope Creek operating license which was issued on July 25, 1986, requires that prior to restart following the first refueling outage, PSE&G shall add four parameters to the Safety Parameter Display System (SPDS) and shall have them operational.

Subsequently, during verification and validation of the current SPDS, PSE&G determined that a new computer system for the SPDS is required to address all the identified deficiencies including the addition of the four parameters required by License Condition 2.C.(13). By letter dated March 19, 1987, PSE&G submitted a report documenting its assessment of the current SPDS. PSE&G met with the NRC on May 21, 1987 to discuss its assessment and its schedule for installation of the new computer system for the SPDS and problems with respect to having the four additional SPDS parameters operational prior to restart following the first refueling outage. It indicated that it was planning to request that the license be modified to extend the scheduler requirement for completing the four additional SPDS parameters.

By letter dated August 18, 1987, PSE&G proposed to amend the license to extend the required date for having these four parameters operational until 90 days after the restart from the first refueling outage or until July 12, 1988, whichever is earlier. Plant shutdown for the first refueling outage is currently scheduled to begin on February 1, 1988.

2.0 EVALUATION

In its August 18, 1987 submittal, PSE&G explained why the four parameters cannot be made operational prior to restart following the refueling outage by describing the sequence of installing and testing the new SPDS system as follows:

"The hardware for the new system will be installed in Hope Creek prior to shutdown for the first refueling. This hardware

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includes a new computer and two new multiplexers. Testing of the communication links between the new and existing multiplexers and the computer and testing of the software will be performed prior to the refueling outage. Connection of the wiring from the field sensors to the new multiplexers cannot be performed at power without possibly affecting safe operation of the plant. These connections will be performed during the outage and will include inputs for the additional parameters in License Condition 2.C.(13). After the new connections are made, circuits checks will be performed to ensure proper operation and that there are no adverse effects on existing plant equipment. Testing of the signals from the eight existing multiplexers will also be performed at this time. This delay of testing until the outage is necessary since testing the new computer at power would inhibit operation of the present SPDS system. Testing of some of the sensors at power could have an adverse effect on plant operation. A method of rapidly transferring operation between computers will be employed during testing to ensure that the existing SPDS can be used immediately if necessary.

Some aspects of the system must be tested while the plant is in operation (e.g., system loading, dynamic time responses). This testing will be conducted during startup, power ascension and while the plant is operating after the outage.

During this testing period following the outage, the present SPDS computer system will be operational and rapidly available. After all testing and debugging of the new computer has been completed, the new SPDS will be placed in operation."

Based on our review of PSE&G's August 18, 1987 submittal, we agree with PSE&G's conclusion that the new SPDS system with the four additional parameters required by License Condition 2.C.(13) cannot be made operational until all of the required testing has been completed. We also agree with PSE&G's conclusion that some of the required testing must be performed while the plant is operating following restart from the outage. We find that the requested 90 day period following restart for completing the testing is reasonable and is necessary in order to assure that the new system is operational. We therefore conclude that the requested change to License Condition 2.C.(13) to extend the schedule required for having the four additional SPDS parameters operational until the earlier of 90 days after restart from the first refueling outage or July 12, 1988 is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released

offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 37551) on October 7, 1987 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: G. Rivenbark

Dated: November 24, 1987

Number 24, 1987

MEMORANDUM FOR: Sholly Coordinator

FROM: Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

SUBJECT: REQUEST FOR PUBLICATION IN MONTHLY FR NOTICE - NOTICE OF
ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Public Service Electric & Gas Company, Docket No. 50-354, Hope Creek

Generating Station, Salem County, New Jersey

Date of application for amendment: August 12, 1987

Brief description of amendment: Revised the Technical Specification to
reduce the emergency diesel generator air starting receiver minimum
required pressure.

Date of issuance: November 24, 1987

Effective date: November 24, 1987

Amendment No. 12

Facility Operating License No. NPF-57. This amendment revised the Technical
Specifications.

Date of initial notice in Federal Register: October 7, 1987 (52 FR 37551)

The Commission's related evaluation of the amendment is contained in a Safety
Evaluation dated November 24, 1987.

No significant hazards consideration comments received: No

Local Public Document Room location: Pennsville Public Library, 190 S.

Broadway, Pennsville, New Jersey 08070

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/s/
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

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WASHINGTON, D. C. 20555
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Broadway, Pennsville, New Jersey 08070

Walter R. Butler
Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II