

February 7, 1985

Docket No. 50-311

Mr. Richard A. Uderitz, Vice President -
Nuclear
Public Service Electric and Gas Company
Post Office Box 236
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Dear Mr. Uderitz:

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Docket File
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The Commission has issued the enclosed Amendment No.29 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated September 29, 1983.

The amendment removes a license condition requiring the installation of upper inspection ports on the Salem Unit No. 2 steam generators.

A copy of the Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/DFischer

Donald Fischer, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No.29 to DPR-75
- 2. Safety Evaluation

cc: w/enclosures
See next page

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Mr. R. A. Uderitz
Public Service Electric & Gas Company

Salem Nuclear Generating Station
Units 1 and 2

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Salem Nuclear Generating Station
Units 1 and 2

- 2 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 29, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:


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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 29, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 29

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise License A as follows:

Remove Page

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Insert Page

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(14) Waterhammer Test (Appendix C, A-1, Supplement 4 and Section 22.2, 11.E.1.1, Supplement 5)

Prior to exceeding 90 percent power, PSE&G shall perform a test program to show that unacceptable waterhammer damage will not result from anticipated feedwater transients to the steam generator. Prior to performing the test program, PSE&G shall obtain NRC approval of the test procedures.

(15) Prior to resuming power operation following the first refueling outage:

(a) Control Rod Guide Thimble (Section 4.2.2, Supplement 4)

PSE&G shall submit the details of the inspection program for control rod guide thimble tube wall wear for NRC approval.

(c) Pressure Isolation Valves (Section 5.3.2, Supplement 5)

PSE&G shall install leak test connections on the pressure isolation valves; until installation of the leak test connections, PSE&G may substitute multiple valve leak tests for Technical Specification 3.4.7.2.f, such that the cumulative leakage from two valves in parallel lines shall not exceed two gallons per minute, and the cumulative leakage from three valves in parallel lines shall not exceed three gallons per minute.

(d) Diesel Generator Reliability (Section 8.3.4, Supplement 5)

PSE&G shall implement the following design and procedural modifications with respect to diesel generator reliability:

- (i) Complete a formal training program for all the mechanical and electrical maintenance and quality control personnel, including supervisors, who are responsible for the maintenance and availability of the diesel generators. The depth and quality of this training program shall be at least equivalent to that of training programs normally conducted by major diesel engine manufacturers.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATION STATION, UNIT NO. 2

DOCKET NO. 50-311

Introduction

On January 27, 1983, the Commission issued Amendment 17 to the Salem, Unit 2, Generating Station Facility Operating License modifying License Condition 2.6-15(b) to allow the licensee to delay the required installation of upper inspection ports in the steam generators until the second refueling outage. By letter dated September 29, 1983 the licensee requested that the requirement to install upper inspection ports be deleted.

Evaluation and Summary

Our original basis for the installation of steam generator upper inspection ports was to facilitate the evaluation and monitoring of the effects of denting in the upper portion of the steam generator tubes. However, experience has shown that visual inspection of the uppermost support plate U-bend transition area of the inner row tubes is no longer necessary. Denting usually takes place initially on the inlet side of the steam generator in the lower part of the tube bundle. The onset denting therefore, could be visually detected earlier in this area through the lower inspection

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ports. In addition, the current state-of-the-art of nondestructive inspection methods such as eddy current testing, profilometry and tube gaging are more effective than visual inspection for the early detection and monitoring of denting. Another factor against the installation of upper inspection ports is that it is desirable to minimize the number of penetrations in the steam generator vessels so as not to provide the opportunity to introduce potential foreign objects into the secondary side. In the future, if degradation takes place in the upper steam generator region and inspection ports are deemed necessary, it can be installed then.

Based on the above, we find that there is no longer a necessity for the installation of upper inspection ports in the absence of active degradation mechanisms in the upper tube bundle. Therefore, we recommend that the licensing condition 2.C.15 (b) be removed from the Salem Generating Station Unit 2, Facility Operating Licensee.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on

such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner,
- and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 7, 1985

Principal Contributor:

H. Conrad