

REGULATORY DIVISION

OCT 10 1980

Docket No.: 50-311

RP/1

Mr. P. L. Mittl, General Manager
Licensing and Environment
Engineering and Construction Department
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Dear Mr. Mittl:

SUBJECT: ISSUANCE OF AMENDMENT NO. 3 TO LICENSE NO. DPR-75 - SALEM NUCLEAR
GENERATING STATION, UNIT NO. 2

The Nuclear Regulatory Commission (the Commission) has issued Amendment No. 3 to License DPR-75 (Enclosure 1) in accordance with two letters from Mr. Frank P. Librizzi, both dated August 22, 1980, requesting changes to the Salem Nuclear Generating Station, Unit No. 2 Appendix A Technical Specifications. An attachment to the letters also provided a safety evaluation to support the changes.

This amendment deletes the requirement for Public Service Electric and Gas Company to plug certain tubes in the steam generators prior to exceeding 5 percent power, approves an organizational change to establish a new Radiation Protection Department, and revises the Technical Specifications concerning High Radiation Area. Copies of our Safety Evaluation and Notice of Issuance of Amendment No. 3 to License No. DPR-75 are enclosed.

Sincerely,

Original signed by
B. C. Buckley

Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

Enclosures:

1. Amendment No. 3 to DPR-75 with Technical Specification page change 5
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

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Handwritten initials and signatures: "M", "BCB for"

OFFICE	DL:LB #3	DL:LB #3	QELD	DL:LB #3	
SURNAME	JLemec	JKernigan	JMoore	FJMiraglia	
DATE	9/25/80	9/25/80	10/1/80	10/10/80	

Mr. R. L. Mittl, General Manager - 2 -

cc: Richard Fryling, Jr., Esq.
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Mr. Leif J. Norrholm
U. S. Nuclear Regulatory Commission
Region I
Drawer I
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Attorney General
Department of Law & Public Safety
State House Annex
Trenton, New Jersey 08625

State House Annex
ATTN: Deputy Attorney General
State of New Jersey
36 West State Street
Trenton, New Jersey 08625

Richard B. McGlynn, Commissioner
Department of Public Utilities,
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

The Honorable Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancock's Bridge, New Jersey 08038

Mr. Bruce Blanchard
Environmental Projects Review
Department of the Interior
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U. S. Environmental Protection Agency
ATTN: Ms. Elizabeth V. Jankus
Office of Environmental Review
Room 2119M, A-104
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Washington, D. C. 20460

EIS Coordinator
U. S. Environmental Protection Agency
Region II Office
26 Federal Plaza
New York, New York 10007

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

President
New Jersey Board of Public Utilities
101 Commerce Street
Newark, New Jersey 07102



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNITS NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The two applications for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 22, 1980 comply with the standards and requirements of the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. This issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 3, are hereby incorporated in the license. Public Service Electric and Gas Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

B. C. Buckley for
Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: OCT 10 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

6-3
6-6
6-18
6-19
8-2

Insert Pages

6-3
6-6
6-18
6-19
8-2

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNITS NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-75

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 - A. The two applications for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 22, 1980 comply with the standards and requirements of the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. This issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

OFFICE
SURNAME
DATE

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 3, are hereby incorporated in the license. Public Service Electric and Gas Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
B. C. Buckley

for Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: OCT 10 1980

OFFICE	DL:LB #3	DL:LB #3	OELD	DL:LB #3
SURNAME	JLee	JKerrigan	JY Moore	FJMiraglia
DATE	9/23/80	9/25/80	10/1/80	10/10/80

ATTACHMENT TO LICENSE AMENDMENT NO. 3
 FACILITY OPERATING LICENSE NO. DPR-75
 DOCKET NO. 50-311

Revise Appendix A as follows:

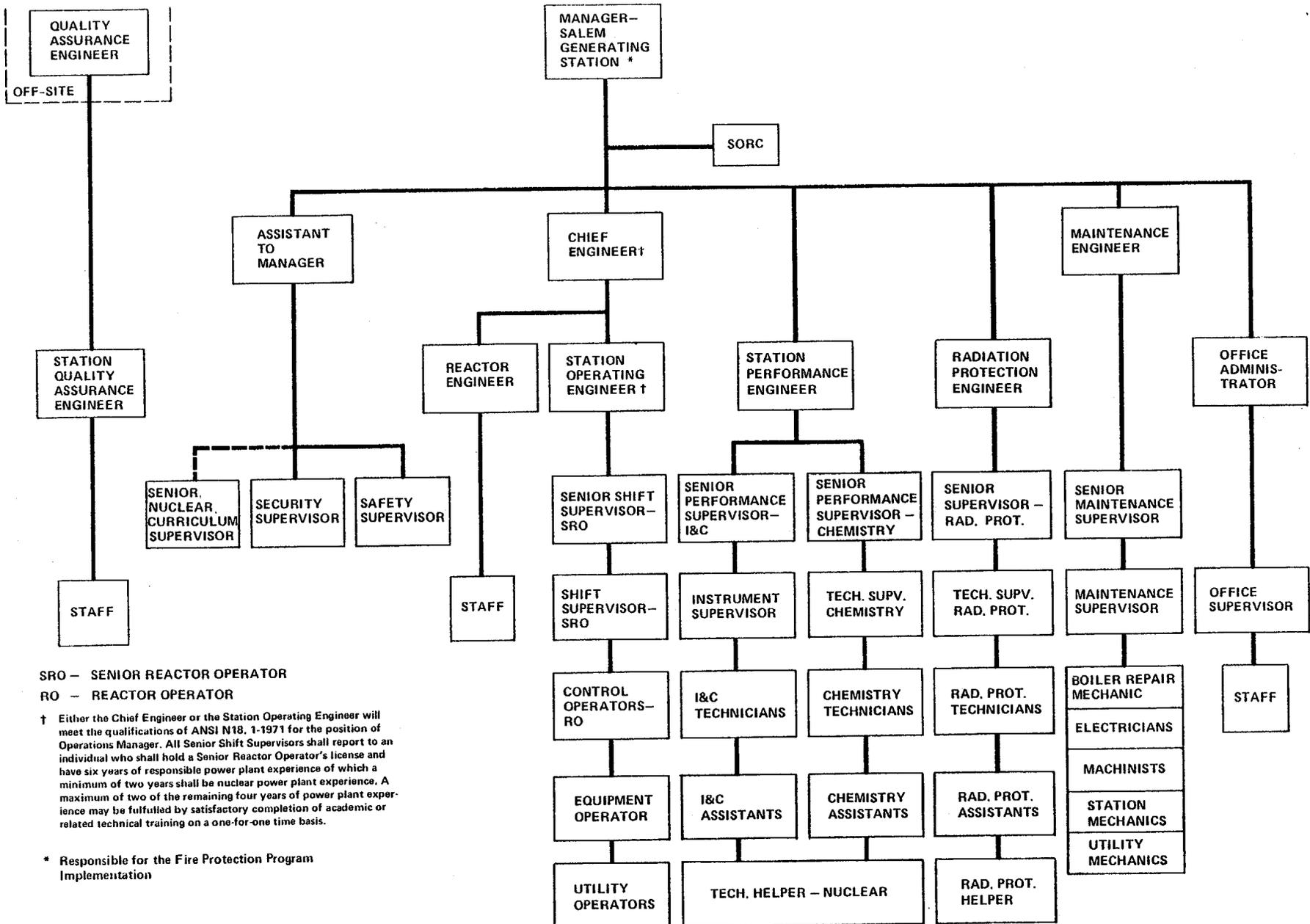
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OFFICE ▶	DL:LB #3	DL:LB #3	DL:LB #3	OELD		
SURNAME ▶	JKerrigan:mec	JLee	FJMiraglia			
DATE ▶	9/ /80	9/ /80	9/ /80	9/ /80		



SRO - SENIOR REACTOR OPERATOR

RO - REACTOR OPERATOR

† Either the Chief Engineer or the Station Operating Engineer will meet the qualifications of ANSI N18. 1-1971 for the position of Operations Manager. All Senior Shift Supervisors shall report to an individual who shall hold a Senior Reactor Operator's license and have six years of responsible power plant experience of which a minimum of two years shall be nuclear power plant experience. A maximum of two of the remaining four years of power plant experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis.

* Responsible for the Fire Protection Program Implementation

FIGURE 6.2.2 FACILITY ORGANIZATION - SALEM GENERATING STATION

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by the Assistant to Manager and maintained under the direction of the Training Engineer and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Member:	Station Operating Engineer
Member:	Station Performance Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior Performance Supervisor - I&C
Member:	Senior Performance Supervisor - Chemistry
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor
Member:	Maintenance Engineer

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

- e. Records of transient or operational cycles for those facility components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SORC and the NRB.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Exposure Permit*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.

*Radiation Protection personnel or personnel escorted by Radiation Protection personnel shall be exempt from the REP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

ADMINISTRATIVE CONTROLS

- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Senior Supervisor - Radiation Protection in the Radiation Exposure Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Senior Shift Supervisor on duty and/or the Senior Supervisor - Radiation Protection.

design basis accidents. Pending completion of such qualification and acceptance by the NRC, each of these detectors shall be replaced at each regularly scheduled refueling outage.

(2) Prior to completion of the second scheduled refueling outage, pressure transmitters and differential pressure transmitters from Barton Lot I shall have been replaced by suitably qualified devices.

(3) Prior to completion of the first refueling outage, the Scotchcast No. 9 resin seals, used at the electrical connection interface on the NAMCO limit switches, will be replaced with Conax Electric Conduction Seal Assemblies.

8.6 Deleted.

8.7 Prior to exceeding five percent power, the licensees shall plug the Row 1 tubes in the steam generators.

8.8 At each of the first three regularly scheduled refueling outages, the licensees shall perform an ultrasonic inspection of the channelhead in the No. 21 steam generator in a selected area.

8.9 Prior to exceeding five percent power, the licensees shall install a reset alarm for the containment purge and pressure-vacuum relief valve reset circuitry.

8.10 Prior to startup following the first regularly scheduled refueling outage, the licensees shall submit the confirmatory results of the containment sump model test program, along with a description of any sump modifications resulting from the tests.

SAFETY EVALUATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 3
TO FACILITY OPERATING LICENSE NO. DPR-75
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
SALEM GENERATING STATION, UNIT NO. 2
DOCKET NO: 50-311

By letters dated August 22, 1980, Public Service Electric and Gas Company (the licensee) submitted requests for an amendment to their Facility Operating License DPR-75 for Salem Generating Station, Unit No. 2. The proposed technical Specification changes deal with the requirement to plug Row 1 tubes in the steam generators, changes to the Salem Radiation Protection organization and rewording of the High Radiation Area section.

The first request is for removal of the requirements of Section 8.7 which requires the licensee to plug Row 1 tubes in the steam generators prior to exceeding 5 percent power. The licensee has requested that the decision to plug Row 1 tubes be delayed until the staff has evaluated the Westinghouse program regarding Row 1 tube cracking. The results are expected in late November or early December. Although the potential for non-denting related Row 1 tube cracking does exist, we have concluded that for the reasons given below, operation of the steam generators without Row 1 being plugged will not constitute an undue risk to the health and safety of the public:

1. The Row 1 tube leaks experienced to date at three operating plants have been small and stable.
2. Primary to secondary leakage rate limits, and associated surveillance requirements will be established to provide assurance that the occurrence of tube cracking during operation will be detected and appropriate corrective action, such as tube plugging, will be taken such that any individual crack present will not become unstable under normal operating, transient or accident conditions.

In addition, the licensee has operated Salem, Unit 1 steam generators beyond the first refueling outage without experiencing any leaking of Row 1 tubes. The design of Unit 2 steam generators is identical to that in Unit 1.

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After the results of the Westinghouse/PGE program become available, we will determine if Row 1 tube plugging will be required at a later date.

The licensee has proposed significant changes to the Salem Radiation Protection organization. PSE&G changes provide for the separation of the radiation protection function from the Performance Department and formation of a new Radiation Protection Department. This new department will be headed by a Radiation Protection Engineer who will report directly to the station Manager. It will have a Senior Supervisor - Radiation Protection (who will act as backup to the Radiation Protection Engineer), Technical Supervisors, Technicians and Technical Assistants, all of whom will be devoted to the function of radiation protection. The remainder of the Performance Department will be modified to split the Technical Assistants such that they are devoted to either the instrumentation and controls function or the chemistry function.

These proposed changes meet our positions in the draft "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 as follows:

1. The Radiation Protection Engineer (RPE - equivalent to the Radiation Protection Manager) reports directly to the Station Manager, independent of operational, technical or administrative groups. The RPE is a required member of the Station Operations Review Committee (SORC). Staff qualifications require that the RPE meet or exceed the recommendations of Regulatory Guide 1.8.
2. The newly formed Radiation Protection Department has an independent radiation protection function at all levels, and is separate from such functions as chemistry. A backup to the RPE, the Senior Supervisor-Radiation Protection has been designated. All Technical Supervisors, Technicians and Technical Assistants within the department are devoted to the radiation protection function.
3. A formal program to replace contractor radiation protection personnel with permanently assigned station radiation protection technicians has been implemented. Additionally, a qualification and retraining program conducted in accordance with ANSI 18.1, provides formal qualification and training for the radiation protection department personnel. PSE&G anticipates the reorganization actions and programs to be fully complete by July 1, 1981. In the interim, a permanent staff is being recruited and all contractor radiation protection technicians are receiving classroom and on the job training on systems, radiological fundamentals and procedures.

These actions and commitments by PSE&G for the Salem Station adequately meet the positions of NUREGs-0660/0694, NUREG-DRAFT "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 regarding Radiation Protection Organization and are therefore satisfactory. An evaluation of the Salem Radiation Protection Department will be performed during a routine inspection.

The final request concerns Section 6.12 High Radiation Area. The proposed Technical Specification change for high radiation area control provides adequate controls for avoiding unnecessary exposure by strictly controlling posting and access. Barricades, High Radiation Area posting, Radiation Exposure Permits, dose rate and dose monitoring, and locking where dose rates exceed 1,000 millirem/hr, are utilized in the Standard Tech Spec. format for High Radiation Area control. This change adequately meets the requirements of 10 CFR Part 20.203(c)(2) and the ALARA considerations of Regulatory Guide 8.8 and is acceptable.

Environmental Consideration

We have determined that this action does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this action is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared.

Conclusion

We have concluded, based on the consideration discussed above, that: (1) because the action does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the action does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and will not be inimical to the common defense and security or to the health and safety of the public.

Dated: OCT 10 1980

SAFETY EVALUATION
 BY THE OFFICE OF NUCLEAR REACTOR REGULATION
 RELATED TO AMENDMENT NO. 3
 TO FACILITY OPERATING LICENSE NO. DPR-75
 PUBLIC SERVICE ELECTRIC AND GAS COMPANY
 SALEM GENERATING STATION, UNIT NO. 2
 DOCKET NO: 50-311

By letters dated August 22, 1980, Public Service Electric and Gas Company (the licensee) submitted requests for an amendment to their Facility Operating License DPR-75 for Salem Generating Station, Unit No. 2. The proposed technical Specification changes deal with the requirement to plug Row 1 tubes in the steam generators, changes to the Salem Radiation Protection organization and rewording of the High Radiation Area section.

The first request is for removal of the requirements of Section 8.7 which requires the licensee to plug Row 1 tubes in the steam generators prior to exceeding 5 percent power. The licensee has requested that the decision to plug Row 1 tubes be delayed until the staff has evaluated the Westinghouse program regarding Row 1 tube cracking. The results are expected in late November or early December. Although the potential for non-denting related Row 1 tube cracking does exist, we have concluded that for the reasons given below, operation of the steam generators without Row 1 being plugged will not constitute an undue risk to the health and safety of the public:

1. The Row 1 tube leaks experienced to date at three operating plants have been small and stable.
2. Primary to secondary leakage rate limits, and associated surveillance requirements will be established to provide assurance that the occurrence of tube cracking during operation will be detected and appropriate corrective action, such as tube plugging, will be taken such that any individual crack present will not become unstable under normal operating, transient or accident conditions.

In addition, the licensee has operated Salem, Unit 1 steam generators beyond the first refueling outage without experiencing any leaking of Row 1 tubes. The design of Unit 2 steam generators is identical to that in Unit 1.

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DATE ▶					

After the results of the Westinghouse/PGE program become available, we will determine if Row 1 tube plugging will be required at a later date.

The licensee has proposed significant changes to the Salem Radiation Protection organization. PSE&G changes provide for the separation of the radiation protection function from the Performance Department and formation of a new Radiation Protection Department. This new department will be headed by a Radiation Protection Engineer who will report directly to the station Manager. It will have a Senior Supervisor - Radiation Protection (who will act as backup to the Radiation Protection Engineer), Technical Supervisors, Technicians and Technical Assistants, all of whom will be devoted to the function of radiation protection. The remainder of the Performance Department will be modified to split the Technical Assistants such that they are devoted to either the instrumentation and controls function or the chemistry function.

These proposed changes meet our positions in the draft "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 as follows:

1. The Radiation Protection Engineer (RPE - equivalent to the Radiation Protection Manager) reports directly to the Station Manager, independent of operational, technical or administrative groups. The RPE is a required member of the Station Operations Review Committee (SORC). Staff qualifications require that the RPE meet or exceed the recommendations of Regulatory Guide 1.8.
2. The newly formed Radiation Protection Department has an independent radiation protection function at all levels, and is separate from such functions as chemistry. A backup to the RPE, the Senior Supervisor-Radiation Protection has been designated. All Technical Supervisors, Technicians and Technical Assistants within the department are devoted to the radiation protection function.
3. A formal program to replace contractor radiation protection personnel with permanently assigned station radiation protection technicians has been implemented. Additionally, a qualification and retraining program conducted in accordance with ANSI 18.1, provides formal qualification and training for the radiation protection department personnel. PSE&G anticipates the reorganization actions and programs to be fully complete by July 1, 1981. In the interim, a permanent staff is being recruited and all contractor radiation protection technicians are receiving classroom and on the job training on systems, radiological fundamentals and procedures.

These actions and commitments by PSE&G for the Salem Station adequately meet the positions of NUREGs-0660/0694, NUREG-DRAFT "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 regarding Radiation Protection Organization and are therefore satisfactory. An evaluation of the Salem Radiation Protection Department will be performed during a routine inspection.

OFFICE ▶						
SURNAME ▶						
DATE ▶						

The final request concerns Section 6.12 High Radiation Area. The proposed Technical Specification change for high radiation area control provides adequate controls for avoiding unnecessary exposure by strictly controlling posting and access. Barricades, High Radiation Area posting, Radiation Exposure Permits, dose rate and dose monitoring, and locking where dose rates exceed 1,000 milli-rem/hr, are utilized in the Standard Tech Spec. format for High Radiation Area control. This change adequately meets the requirements of 10 CFR Part 20.203(c)(2) and the ALARA considerations of Regulatory Guide 8.8 and is acceptable.

Environmental Consideration

We have determined that this action does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this action is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared.

Conclusion

We have concluded, based on the consideration discussed above, that: (1) because the action does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the action does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and will not be inimical to the common defense and security or to the health and safety of the public.

Dated: OCT 10 1980

DL:LB #3 for DL:LB #3

OFFICE	DL:LB #3	DL:LB #3			
SURNAME	JDKerrigan:mec	FJMiraglia			
DATE	9/25/80	10/10/80			

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO: 50-311

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 3 to License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 2 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment deletes the requirement for Public Service Electric and Gas Company to plug certain tubes in the steam generators prior to exceeding 5 percent power, approves an organizational change to establish a new Radiation Protection Department, and revises the Technical Specification concerning High Radiation Area.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

OFFICE ▶

SURNAME ▶

DATE ▶

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the two letters of application for amendment both dated August 22, 1980, (2) Amendment No. 3 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of October, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

[Signature]
B. C. Buckley, Acting Chief
Licensing Branch No. 3
Division of Licensing

See Previous Concurrences

OFFICE	DL:LB 3	DL:LB 3	OELD	DL:LB 3	
SURNAME	JLise	JKerrigan		FJMiraglia	
DATE	10/10/80	9/25/80	10/1/80	10/10/80	

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the two letters of application for amendment both dated August 22, 1980, (2) Amendment No. 3 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this day of

FOR THE NUCLEAR REGULATORY COMMISSION

Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

OFFICE ▶	DL:LB.#3	DL:LB.#3	OELD	DL:LB.#3		
SURNAME ▶	JLde:mec	JKerrigan	J Moore	FJMiraglia		
DATE ▶	9/23/80	9/25/80	10/1/80	10/1/80		