

December 8, 1980

Docket No.: 50-311

Mr. R. L. Mittl, General Manager
Licensing and Environment
Engineering and Construction Department
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Dear Mr. Mittl:

SUBJECT: ISSUANCE OF AMENDMENT NO. 4 TO LICENSE NO. DPR-75 - SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

The Nuclear Regulatory Commission (the Commission) has issued Amendment No. 4 to License DPR-75 (Enclosure 1) in accordance with a letter from Mr. Frank P. Librizzi, dated November 20, 1980, requesting changes to the Salem Nuclear Generating Station, Unit No. 2 Technical Specifications, Appendices A and B. An attachment to the letter also provided a safety evaluation to support the changes.

This amendment approves an organizational change to the Salem operation and support staffs and revises related Technical Specifications. Copies of our Safety Evaluation and Notice of Issuance of Amendment No. 4 to License No. DPR-75 are enclosed.

Sincerely,

/s/

Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

Enclosures:

1. Amendment No. 4 to DPR-75 with Technical Specification page changes
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

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396
 Concern as to subject to understanding that no prior expression of interest in a hearing had been received

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DATE	12/2/80	12/1/80	12/4/80	12/5/80	12/1/80	12/8/80

Mr. R. L. Mittl

- 2 -

December 8, 1980

cc: Richard Fryling, Jr., Esq.
Assistant General Counsel
Public Service Electric & Gas Company
80 Park Place
Newark, New Jersey 07100

Mark Wetterhahn, Esq.
Conner, Moore & Corber
Suite 1050
1747 Pennsylvania Avenue, N. W.
Washington, D. C. 20006

Mr. Leif J. Norrholm
c/o U. S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, New Jersey 08038

Attorney General
Department of Law & Public Safety
State House Annex
Trenton, New Jersey

State House Annex
ATTN: Deputy Attorney General
State of New Jersey
36 West State Street
Trenton, New Jersey 08625

Mr. Richard B. McGlynn, Commissioner
Department of Public Utilities,
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

President
New Jersey Board of Public Utilities
101 Commerce Street
Newark, New Jersey 07102

The Honorable Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancock's Bridge, New Jersey 08038

Mr. Bruce Blanchard
Environmental Projects Review
Department of the Interior
Room 4256
18th and C Street, N.W.
Washington, D.C. 20240

U.S. Environmental Protection Agency
ATTN: Ms. Elizabeth V. Jankus
Office of Environmental Review
Room 2119M, A-104
401 M Street, S.W.
Washington, D.C. 20460

EIS Coordinator
U.S. Environmental Protection Agency
Region II Office
26 Federal Plaza
New York, New York 10007

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U.S. Environmental Protection Agency
Washington, D.C. 20460

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Action	File	Note and Return
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As Requested	For Correction	Prepare Reply
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REMARKS

You should review your correspondence files to determine whether there are any outstanding requests for hearing or expressions of interest in connection with this matter. If there are any, please discuss with me or Mr. Christenbury before taking action.

Based on survey of license amendment actions dated Nov 25, 1980 (R Tedesco to T Novak) No correspondence indicating requests for hearing or expressions of interest in this matter are outstanding.

*A. Miraglia
12/5/80*

DO NOT use this form as a RECORD of approvals, concurrences, disposals, clearances, and similar actions

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Joe Scinto

5041-102

OPTIONAL FORM 41 (Rev. 7-76)
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FPMR (41 CFR) 101-11.206

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*Jean,
Attach this to Yellow after final distribution is made.
A. CP 1*



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNITS NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4
License No. DPR-75

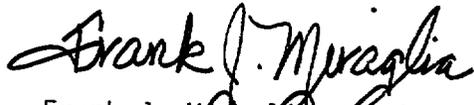
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated November 20, 1980 complies with the standards and requirements of the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 4, are hereby incorporated in the license. Public Service Electric and Gas Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frank J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 8, 1980

December 8, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 4

FACILITY LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

~~6-2~~
~~6-3~~
~~6-6~~
~~6-7~~
~~6-8~~
~~6-9~~
~~6-11~~
~~6-12~~

Insert Pages

~~6-2~~
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Revise Appendix B as follows:

Remove Page

~~5-1~~

Insert Page

~~5-1~~

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. A health physics technician shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by a licensed Senior Reactor Operator who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times[#]. The Fire Brigade shall not include 4 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

[#]Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

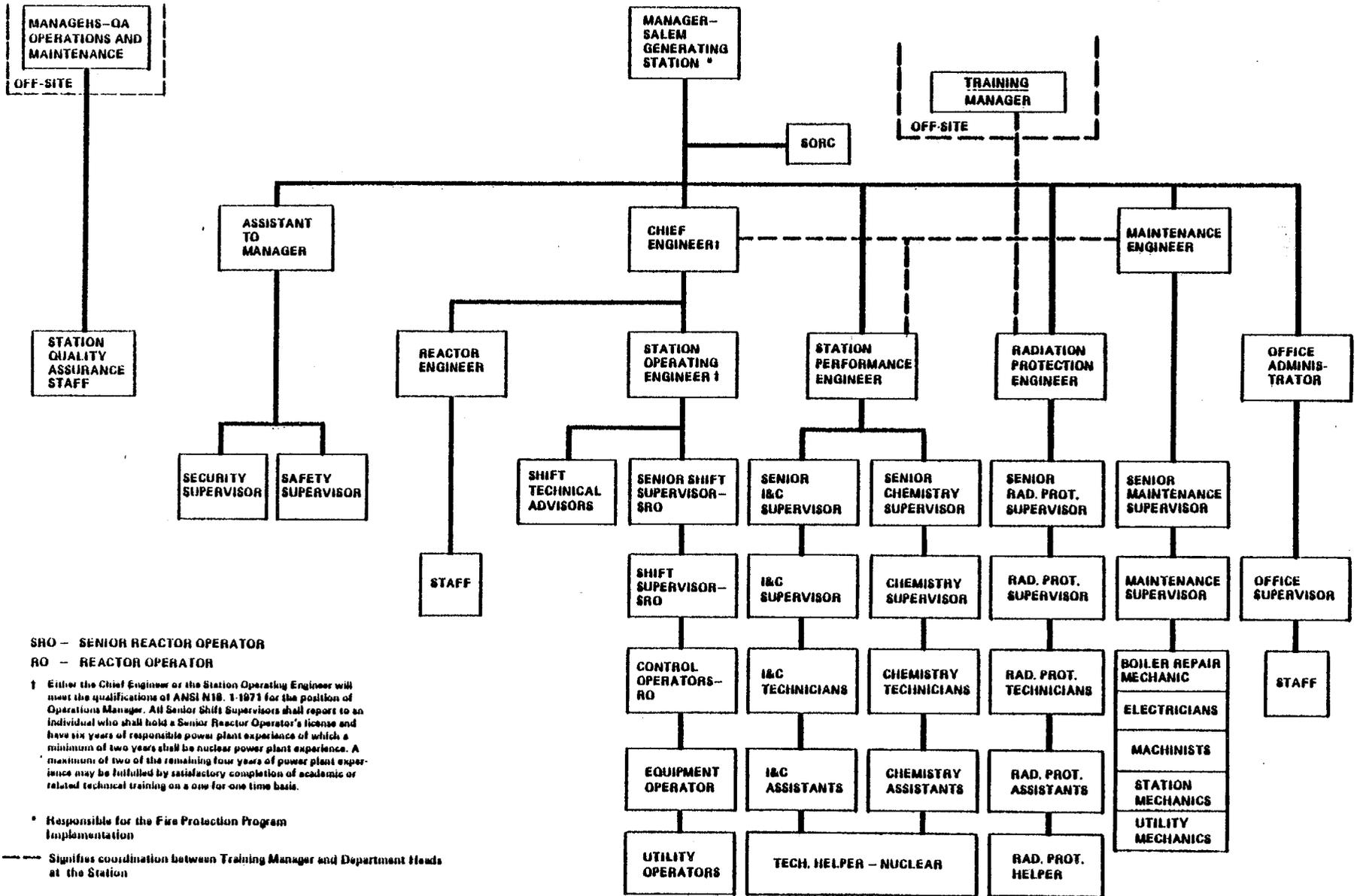


FIGURE 6.2.2 FACILITY ORGANIZATION-SALEM GENERATING STATION

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION[#]

Condition of Unit 2 - No Fuel in Unit 1

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	2	1*
OL	2	1
Non-Licensed	2	1

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL	4	3***
Non-Licensed	4	3

Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL	3***	2
Non-Licensed	3	3

TABLE 6.2-1 (Continued)

*Does not include the licensed Senior Reactor Operator supervising CORE ALTERATIONS.

**Each unit will be supervised by a shift supervisor who is a licensed SRO on that unit; or by a single individual if he is licensed on both units. The second senior operator licensed for each unit must be stationed in the control room area at all times when the unit is in operating modes 1 through 4; and could also be a single individual if he is licensed on Units 1 and 2.

***A reactor operator licensed for each unit must be at the controls of that unit at all times when fuel is in the reactor. Also, a relief reactor operator licensed for each unit must be available on-shift. This could be a single individual if he is licensed for both units.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineers who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Vice Chairman:	Station Performance Engineer
Vice Chairman:	Maintenance Engineer
Member:	Station Operating Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior I & C Supervisor
Member:	Senior Chemistry Supervisor
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.

ADMINISTRATIVE CONTROLS

AUTHORITY

- 6.5.1.7 The Station Operations Review Committee shall:
- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the General Manager-Nuclear Production and the Nuclear Review Board of disagreement between the SORC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Station Operations Review Committee shall maintain written minutes of each meeting that, at a minimum, document the results of all SORC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the General Manager-Nuclear Production and Chairman of the Nuclear Review Board.

6.5.2 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

- 6.5.2.1 The Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:
- a. nuclear power plant operations
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The NRB shall be composed of the:

Chairman:	Assistant Vice President - Production
Vice Chairman:	Assistant to General Manager-Fuel Supply
Member:	General Manager-Licensing and Environment
Member:	Manager-Nuclear Operations Support
Member:	General Manager-Corporate QA
Member:	General Manager-Nuclear Production
Member:	Manager-Salem Generating Station
Member:	Principal Engineer
Member:	Manager-Hope Creek Generating Station

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.2.5 The NRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 The minimum quorum of NRB necessary for the performance of the NRB review and audit functions of these technical specifications shall consist of the Chairman or his designated alternate and at least 4 NRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.7 The NRB shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

ADMINISTRATIVE CONTROLS

- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this operating license.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Station Operations Review Committee.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the NRB. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.

ADMINISTRATIVE CONTROLS

- g. Any other area of facility operation considered appropriate by the NRB or the Vice President-Production.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.2.9 The NRB shall report to and advise the Vice President-Production on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President-Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President-Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President-Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SORC and submitted to the NRB and the General Manager-Nuclear Production.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the General Manager- Nuclear Production and to the NRB within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the General Manager-Nuclear Production within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the SORC and approved by the Station Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

5. ADMINISTRATIVE CONTROLS

Administrative controls below apply to Appendix B - Part II only.

5.1 Responsibility

5.1.1 The implementation of the surveillance programs external to the plant, including sampling, sample analysis, evaluation of results and the preparation of required reports is the responsibility of the Licensing and Environment Department. This Department is responsible for the assignment of personnel to the above functions, for assurance that appropriate written procedures are utilized in the surveillance program activities and for assuring the quality of surveillance program results.

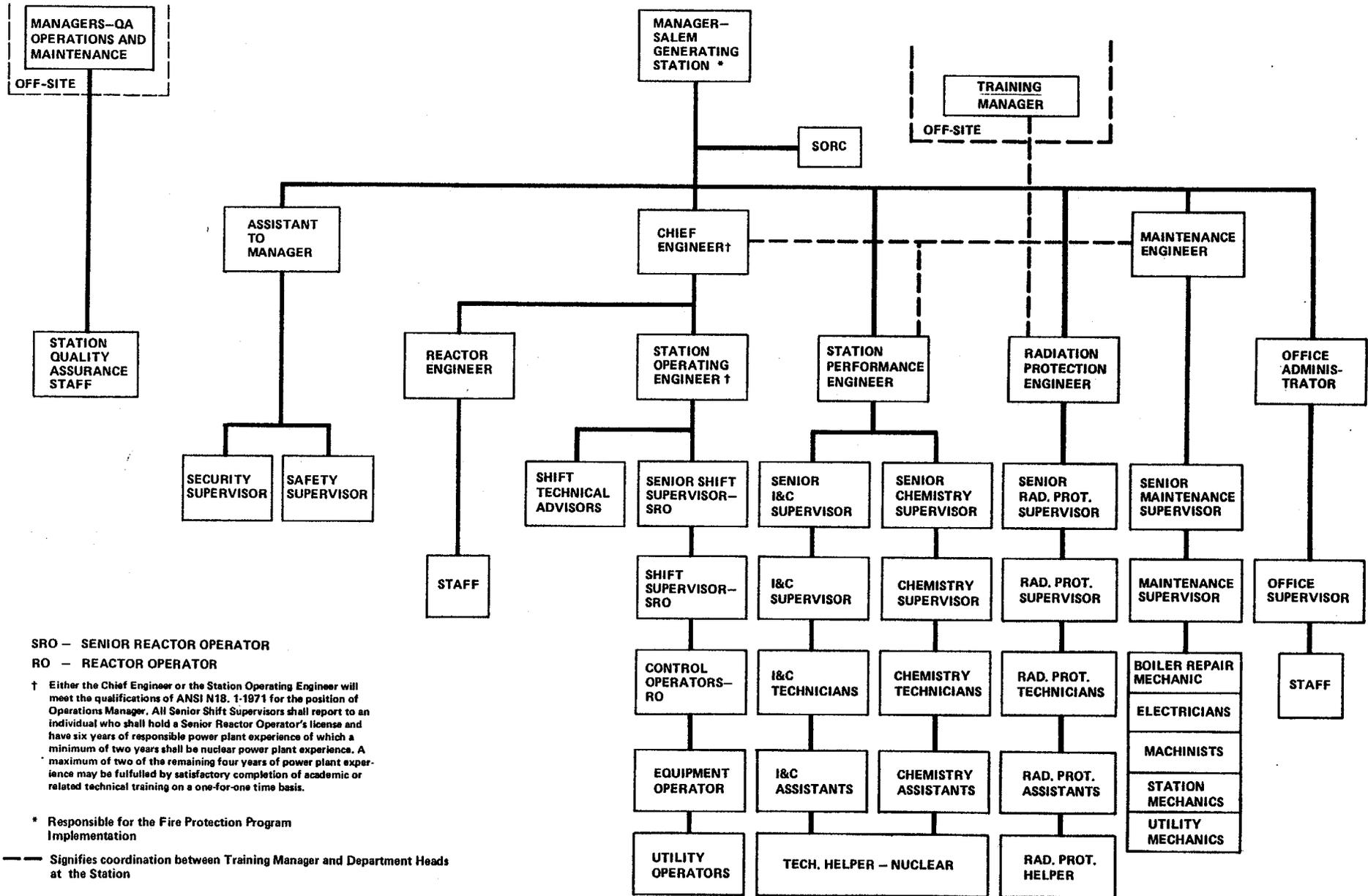
5.1.2 The Station Manager or his delegated alternate is responsible for operating the plant in compliance with the limiting conditions for operation as specified in the Environmental Technical Specifications and for the in-plant monitoring necessary to ensure such operation. His responsibility includes assurance that plant activities are conducted in such a manner as to provide continuing protection to the environment and that personnel performing such activities use appropriate written procedures as described in Section 5.5.

5.2 Review and Audit

5.2.1 Nuclear Review Board (NRB)

The NRB shall have the following responsibilities concerning the environmental impact of the plant:

1. NRB shall review:
 - a. Proposed Environmental Technical Specifications changes or license amendments.
 - b. Violations of Environmental Technical Specifications.
 - c. Environmental Monitoring Program and Evaluations.
 - d. Routine and non-routine reports required by the Environmental Technical Specifications.
2. The NRB shall, at least once each year, conduct (or cause to have conducted) and evaluate audits of:
 - a. Plant operation to assure Environmental Technical Specification compliance.



SRO - SENIOR REACTOR OPERATOR
 RO - REACTOR OPERATOR

† Either the Chief Engineer or the Station Operating Engineer will meet the qualifications of ANSI N18.1-1971 for the position of Operations Manager. All Senior Shift Supervisors shall report to an individual who shall hold a Senior Reactor Operator's license and have six years of responsible power plant experience of which a minimum of two years shall be nuclear power plant experience. A maximum of two of the remaining four years of power plant experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis.

* Responsible for the Fire Protection Program Implementation

--- Signifies coordination between Training Manager and Department Heads at the Station

FIGURE 6.2.2 FACILITY ORGANIZATION-SALEM GENERATING STATION

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Vice Chairman:	Station Performance Engineer
Vice Chairman:	Maintenance Engineer
Member:	Station Operating Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior I & C Supervisor
Member:	Senior Chemistry Supervisor
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Station Operations Review Committee shall:

- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the General Manager-Nuclear Production and the Nuclear Review Board of disagreement between the SORC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Station Operations Review Committee shall maintain written minutes of each meeting that, at a minimum, document the results of all SORC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the General Manager-Nuclear Production and Chairman of the Nuclear Review Board.

6.5.2 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.2.1 The Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The NRB shall be composed of the:

Chairman:	General Manager-Electric Production
Vice Chairman:	Assistant to General Manager-Fuel Supply
Member:	Assistant Manager-Licensing and Environment
Member:	Assistant Manager Nuclear Operations Support
Member:	General Manager Vice Corporate QA
Member:	General Manager Nuclear Production
Member:	Manager-Salem General Station
Member:	Principal Engineer
Member:	Manager-Hope Creek Generating Station

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.2.5 The NRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 The minimum quorum of NRB necessary for the performance of the NRB review and audit functions of these technical specifications shall consist of the Chairman or his designated alternate and at least 4 NRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.7 The NRB shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

ADMINISTRATIVE CONTROLS

- g. Any other area of facility operation considered appropriate by the NRB or the Vice President-Production.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.2.9 The NRB shall report to and advise the Vice President-Production on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President-Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President-Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President-Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SORC and submitted to the NRB and the General Manager-Nuclear Production.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the General Manager- Nuclear Production and to the NRB within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the General Manager-Nuclear Production within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the SORC and approved by the Station Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

5. ADMINISTRATIVE CONTROLS

Administrative controls below apply to Appendix B - Part II only.

5.1 Responsibility

5.1.1 The implementation of the surveillance programs external to the plant, including sampling, sample analysis, evaluation of results and the preparation of required reports is the responsibility of the Licensing and Environment Department. This Department is responsible for the assignment of personnel to the above functions, for assurance that appropriate written procedures are utilized in the surveillance program activities and for assuring the quality of surveillance program results.

5.1.2 The Station Manager or his delegated alternate is responsible for operating the plant in compliance with the limiting conditions for operation as specified in the Environmental Technical Specifications and for the in-plant monitoring necessary to ensure such operation. His responsibility includes assurance that plant activities are conducted in such a manner as to provide continuing protection to the environment and that personnel performing such activities use appropriate written procedures as described in Section 5.5.

5.2 Review and Audit

5.2.1 Nuclear Review Board (NRB)

The NRB shall have the following responsibilities concerning the environmental impact of the plant:

1. NRB shall review:
 - a. Proposed Environmental Technical Specifications changes or license amendments.
 - b. Violations of Environmental Technical Specifications.
 - c. Environmental Monitoring Program and Evaluations.
 - d. Routine and non-routine reports required by the Environmental Technical Specifications.
2. The NRB shall, at least once each year, conduct (or cause to have conducted) and evaluate audits of:
 - a. Plant operation to assure Environmental Technical Specification compliance.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 4
TO FACILITY OPERATING LICENSE NO. DPR-75
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
SALEM GENERATING STATION, UNIT NO. 2
DOCKET NO: 50-311

As a result of a proposed management reorganization for the Salem Generating Station, the licensee has requested changes to Section 6.0 - "Administrative Controls" of Appendix A (Safety) of the Technical Specifications. These proposed changes also affect Section 5.0 - "Administrative Controls" of Appendix B (Environmental) of the Technical Specifications.

Under the organization in effect at the time Facility Operating License DPR-75 was issued for Salem Unit 2, all of the Company's operating plants reported directly to the General Manager - Electric Production who, in turn, reported to the Vice President - Production, and who was simultaneously responsible for both fossil and nuclear operations. We commented in our Safety Evaluation Report Supplement that this organizational arrangement could tend to dilute the attention given by corporate management to nuclear operations, but we concluded that the organization was workable due to the heavy emphasis placed on nuclear operations.

The licensee now proposes to divide the duties previously exercised by the General Manager - Electric Production, by establishing a General Manager - Nuclear Production, a General Manager - Fossil Production, and a General Manager - Production Support. The Company's nuclear plants will now report to the General Manager - Nuclear Production who will also direct the activities of a newly designated Manager - Emergency Preparedness and a Manager - Nuclear Operations Support, previously designated the Manager - Nuclear Operations. The new General Manager - Production Support will control most of the corporate level technical resources supporting the Company's operating plants. This proposed new organizational arrangement is shown on the attached Figure 6.2-1.

Our evaluation indicates that this proposed new organizational arrangement is superior to the previous arrangement since the new General Manager - Nuclear Production can devote his entire attention to the nuclear plant operation. Further, the Vice President - Production can now look to the General Manager - Production Support for most of the technical resources needed to support safe plant operation. Also, the revised organizational structure provides a Manager - Emergency Preparedness who can devote his attention to the emergency planning, security and fire protection functions at the nuclear plants. As shown in the revised Figure 6.2-1, the Manager - Emergency Preparedness will have direct access to the Vice President - Production on emergency preparedness matters.

The proposed revised organization also provides an Assistant Vice President - Production, who supervises the Manager - Personnel Affairs and Administration, and through him the corporate Training Manager. The Assistant Vice President - Production also will serve as the Chairman of the Nuclear Review Board.

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Under the organization in being at the time Salem Unit 2 was licensed, the Manager - Quality Assurance reported to the Vice President - Engineering and Construction. In the new proposed organization, a General Manager - Corporate Quality Assurance has been created, on a reporting level comparable to the Vice President - Engineering and Construction and the Vice President - Production. This is shown on the attached Figure 6.2-1. This change assures the continued separation of the quality assurance function from the production function and, in fact, should result in greater corporate attention being given to quality assurance. We conclude that it is acceptable.

Figure 6.2-2, attached, depicts the proposed new station organization. There are a number of significant changes that the licensee proposes to make. In addition to a number of title changes, the Radiation Protection Engineer has now been moved such that he is free from plant operational pressures and has direct access to the Station Manager. This is in accord with the desires of the NRR Radiation Assessment Branch and is acceptable. The Senior Training Supervisor at the Station has been replaced by a corporate level Training Manager, who coordinates training activities directly with the Department Heads at the Station. This should result in increased emphasis on training and is acceptable.

At the time the license for Unit 2 was issued, the licensee had the Shift Technical Advisors reporting to the Reactor Engineer. By letter dated October 29, 1980, the licensee informed us that, in order to obtain a greater involvement of the STAs in shift operation and an improved awareness of plant conditions, the STAs had been shifted so that they would report to the Operating Engineer rather than the Reactor Engineer. This change is reflected on the attached Figure 6.2-2. This reporting arrangement maintains STA independence from the actual shift operating crew and is acceptable.

Figure 6.2-2 also shows the Station Quality Assurance Staff now reporting to the Manager - QA Operations and Maintenance at the Corporate level who, in turn, reports to the new General Manager - Corporate Quality Assurance, as shown on Figure 6.2-1. This maintains the independence of Station QA activities from operating pressures and is acceptable.

As a result of the organizational and title changes as noted above, the text of Section 6.0 of the Safety Technical Specifications and Section 5.0 of the Environmental Technical Specifications has been altered. We have reviewed these text changes and find them to be in accordance with the proposed new organization.

Overall, our review indicates that the proposed organizational changes covered by the requested Technical Specification amendment should result in strengthened management controls of the nuclear plant operations, as compared to the organization that was in effect at the time the operating license for Salem Unit 2 was issued. On this basis, we conclude that the proposed management reorganization is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 8, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO: 50-311

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 4 to License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 2 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment approves an organizational change to the Salem operation and support staffs, and revises the related Technical Specifications.

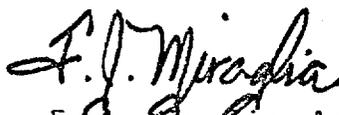
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 20, 1980, (2) Amendment No. 4 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 8th day of December, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



F. J. Miraglia, Acting Chief
Licensing Branch No. 3
Division of Licensing