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OCT 15 1981

Docket No. 50-311

Mr. F. W. Schneider, Vice President
 Production
 Public Service Electric and Gas Company
 80 Park Plaza 15A
 Newark, New Jersey 07101



Dear Mr. Schneider:

The Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated September 23, 1981.

The amendment removes the limit of 90 hours per year for the use of the Pressure-Vacuum Relief portion of the Containment Ventilation System.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Gary C. Meyer, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

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Enclosures:

1. Amendment No. 2 to DPR-75
2. Safety Evaluation
3. Notice of Issuance

ccw/enclosures:
 See next page

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FR NOTICE
 & AMENDMENT
 ONLY

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Mr. F. W. Schneider
Public Service Electric and Gas Company

cc: Mark J. Wetterhahn, Esquire
Conner, Moore and Corber
Suite 1050
1747 Pennsylvania Avenue, NW
Washington, D. C. 20006

Richard Fryling, Jr., Esquire
Assistant General Solicitor
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Gene Fisher, Bureau of Chief
Bureau of Radiation Protection
380 Scotch Road
Trenton, New Jersey 08628

Mr. Henry J. Midura, Manager
Salem Nuclear Generating Station
Public Service Electric and Gas Company
P. O. Box 168
Hancocks Bridge, New Jersey 08038

Salem Free Library
112 West Broadway
Salem, New Jersey 08079

Leif J. Norrholm, Resident Inspector
Salem Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, New Jersey 08038

Attorney General
Department of Law and Public Safety
State House Annex
Trenton, New Jersey 08625

Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancocks Bridge, New Jersey 08038

Richard B. McGlynn, Commissioner
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

Deputy Attorney General
State House Annex
State of New Jersey
36 West State Street
Trenton, New Jersey 08625

Regional Radiation Representative
EPA Region II
26 Federal Plaza
New York, New York 10007

Mr. R. L. Mittl, General Manager
Licensing and Environment
Public Service Electric and Gas Company
80 Park Plaza
Newark, New Jersey 07101

John M. Zupko, Jr., Manager
Nuclear Operations Support
Public Service Electric and Gas Company
80 Park Plaza 15-A
Newark, New Jersey 07101

Lower Alloways Creek Township
c/o Michael C. Facemeyer, Clerk
Municipal Building
Hancocks Bridge, New Jersey 08038

Mr. Alfred C. Coleman, Jr.
Mrs. Eleanor G. Coleman
35 K Drive
Pennsville, New Jersey 08070

Mr. F. W. Schneider
Public Service Electric and Gas Company

cc: Mr. R. A. Uderitz, General Manager
Nuclear Production
Production Department
Public Service Electric and Gas
Company
80 Park Plaza 15A
Newark, New Jersey 07101

Mr. Mark L. First
Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
Environmental Protection Section
36 West State Street
Trenton, New Jersey 08625

Mr. Dale Bridenbaugh
M.H.B. Technical Associates
1723 Hamilton Avenue, Suite K
San Jose, California 95125

Mr. J. T. Boettger, General Manager
Quality Assurance I&E
Public Service Electric and Gas
Company
80 Park Plaza
Newark, New Jersey 07101

Mr. Edwin A. Liden, Manager
Nuclear Licensing
Licensing and Environment Dept.
80 Park Plaza 16D
Newark, New Jersey 07101

Carl Valore, Jr., Esquire
Valore, McAllister, Aron and
Westmoreland, P.A.
535 Tilton Road
Northfield, New Jersey 08225

June D. MacArtor, Esquire
Deputy Attorney General
Tatnall Building
P. O. Box 1401
Dover, Delaware 19901

Keith A. Orsdorff, Esquire
Department of the Public Advocate
Division of Public Interest Advocacy
520 East State Street
Trenton, New Jersey 08601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY AND ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 2
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 23, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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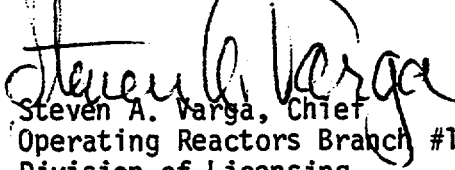
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 2, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 15, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 2

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

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Insert Page

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CONTAINMENT SYSTEMS

CONTAINMENT VENTILATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.1.7 The containment purge supply and exhaust isolation valves* shall be closed.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one containment purge supply and/or exhaust isolation valve open, close the open valve(s) within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.7 The containment purge supply and exhaust isolation valves shall be determined closed at least once per 31 days.

*The containment pressure-vacuum relief isolation valves may be opened on an intermittent basis, under administrative control, as necessary to satisfy the requirement of Specification 3.6.1.4.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILDELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

DOCKET NO. 50-311

Introduction

In Supplement 4 to the Salem Unit 2 Safety Evaluation Report we concluded, with regards to the Salem Unit 2 Containment Isolation System, that:

- (1) The system is acceptable in terms of valve operability since (a) the valves in the 36-inch lines will be closed during all plant operations except for refueling and cold shutdown, (b) the valves in the 10-inch line will be aligned such that the maximum open position corresponds to 60 degrees, and (c) the torque required to close the valves in the 10-inch line, in the event of an accident, is well below the allowable actuator torque;
- (2) The use of the pressure-vacuum relief system, as needed, during operating modes requiring containment integrity, namely startup, power operation, hot standby or hot shutdown, is acceptable since the combined loss-of-coolant accident dose is within the guideline values of 10 CFR Part 100;
- (3) Debris screens are satisfactorily installed;
- (4) The reset circuitry is acceptable in that it will prevent inadvertent reset of the containment isolation signal;
- (5) The containment purge system and pressure-vacuum relief system designs satisfy the provisions of Branch Technical Position CSB 6-4 and that operation of the system as proposed is acceptable.

In addition to the above findings, we referenced our review of the long-term operability of the containment purge valves and stated that any changes to our requirements identified through our review would be imposed on Salem Unit 2, as appropriate. In Supplement 5 to the Salem Unit 2 Safety Evaluation Report the long-term valve operability review remained an open item. However, a

limitation was imposed on the use of the pressure-vacuum relief system to a total of no more than 90 hours per year during normal plant operating modes of startup, power, hot standby, and hot shutdown.

By letter, dated September 23, 1981, the licensee requested a change to the Salem Unit 2 Safety Technical Specifications (Appendix A) to remove the present limit of 90 hours per year for the use of the pressure-vacuum relief system.

Discussion and Evaluation

By letter, dated November 28, 1978, the Commission (NRC) requested all licensees of operating reactors to respond to generic concerns about containment purging or venting during normal plant operation. The review of the licensee's response to this letter was subsequently interrupted by the TMI accident and its demand on staff resources. Consequently, an Interim Position on containment purging and venting was transmitted to the licensee for Salem Unit 1 on October 30, 1979. In response to this the licensee implemented specific short-term corrective actions to remain in effect pending completion of the longer-term review of the licensee's response to our November 28, 1978 letter.

The NRC Interim Position transmitted on October 30, 1979, was used in the review of Salem Unit 2 for issuance of its full power license. The results of this review, as documented in Supplement 4 to the Salem 2 SER and discussed above, show that Salem Unit 2 satisfies the October 30, 1979 interim position and is satisfactory for operation pending completion of our long-term review. Subsequent to this finding, a further interim limitation of 90 hours per year was placed on the use of the pressure-vacuum relief system at Salem Unit 2 during normal operation.

This 90 hour per year limitation has been subsequently refined to specify that this limit shall be imposed when the estimated amount of radioactivity released during the time required to close the valves, following a LOCA, causes the total dose to exceed the Guidelines of 10 CFR Part 100. As stated in supplement 4 to the Salem Unit 2 SER and reaffirmed in the licensee's letter of September 23, 1981 the radiological consequences of a LOCA while using the pressure-vacuum relief system is within the guideline values of 10 CFR Part 100.

Safety Conclusion

Based on the above discussion we conclude that Salem Unit 2 satisfies our Interim Position on containment purging and venting and shall operate under those guidelines pending completion of our long-term review. The 90 hour per year limitation currently in place may be deleted and an administrative goal established which represents a limit on the annual hours of purging expected.

The licensee, in its letter of September 23, 1981, has suggested that a goal of 2,000 hours per year be established, for the use of the pressure-vacuum relief system at Salem Unit 2. The licensee has not provided a bases for this goal. We shall require the licensee to provide this bases prior to January 15, 1982, for our review and determination as to its acceptability. At that time the goal would be placed in the Technical Specification Bases. We now require that the 36-inch valves should be locked closed when above 200°F reactor coolant temperature and verified locked closed every 31 days; and the 10-inch valves shall be operated within the NRC Staff Interim Position pending our evaluation of the long term operability review. It is emphasized that our Interim Position requires that purging/venting be minimized and conducted only when justified for safety purposes.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 15, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-311PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 2 to Facility Operating License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 2 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment removes the limit of 90 hours per year for the use of the Pressure-Vacuum Relief portion of the Containment Ventilation System.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

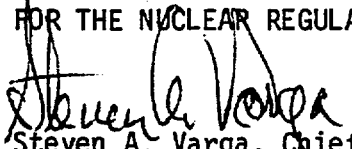
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 23, 1981, (2) Amendment No. 2 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 15th day of October, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing