Docket No. 50-311 May 30, 1985

DISTRIBUTION

| | |
|---|---|
| Docket File | NRC PDR |
| L PDR | ORB#1 Rdg |
| HThompson | DFischer |
| CParrish | OELD |
| SECY | LHarmon |
| EJordan | BGrimes |
| JPartlow | TBarnhart 4 |
| WJones | EButcher |
| ACRS 10 | CMiles |
| RDiggs | RBallard |
| Gray File | KDempsey TQuay |

Mr. C. A. McNeill, Jr.
Vice President - Nuclear
Public Service Electric and Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. McNeill:

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated September 28, 1983 and supplemented November 21, 1984.

The amendment revises the control room leak test pressure from 1/4 inch W.G. to 1/8 inch W.G. for surveillance testing.

A copy of the Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/DFischer

Donald Fischer, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 37 to DPR-75
2. Safety Evaluation

cc: w/enclosures
See next page

| ORB#1:DL | ORB#1:DL | BC-ORB#1:DL | OELD | AD:OR:DL |
|---|---|---|---|---|
| CParrish | DFischer;ps | SVarga | Telephone cc DCF (w/o JM) | GLainas |
| 5/16/85 | 5/16/85 | 5/17/85 | 5/30/85 | 5/30/85 |

8506200415 850530
PDR ADOCK 05000311
P PDR

cc: Mark J. Wetterhahn, Esquire
Conner and Wetterhahn
Suite 1050
1747 Pennsylvania Avenue, NW
Washington, DC 20006

Richard Fryling, Jr., Esquire
Assistant General Solicitor
Public Service Electric & Gas Company
P. O. Box 570 - Mail Code T5E
Newark, New Jersey 07101

Gene Fisher, Bureau of Chief
Bureau of Radiation Protection
380 Scotch Road
Trenton, New Jersey 08628

Mr. John M. Zupko, Jr.
General Manager - Salem Operations
Public Service Electric & Gas Company
Post Office Box E
Hancocks Bridge, New Jersey 08038

Robert Traae, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancocks Bridge New Jersey 08038

Thomas Kenny, Resident Inspector
Salem Nuclear Generating Station
U.S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, New Jersey 08038

Richard F. Engel
Deputy Attorney General
Department of Law and Public Safety
CN-112
State House Annex
Trenton, New Jersey 08625

Mr. Anthonly J. Pietrofitta
General Manager
Power Production Engineering
Atlantic Electric
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Richard B. McGlynn, Commission
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

Mr. R. L. Mittl, General Manager
Nuclear Assurance and Regulation
Public Service Electric & Gas Co.
Mail Code T16D - P. O. Box 570
Newark, New Jersey 07101

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, New Jersey 08038

Mr. Edwin A. Liden, Manager
Nuclear Licensing & Regulation
Public Service Electric & Gas Company
Hancocks Bridge, New Jersey 08038

Mr. Charles P. Johnson
General Manager
Nuclear Quality Assurance
Public Service Electric & Gas Company
Hancocks Bridge, New Jersey 08038

Mr. David Wersan
Assistant Consumer Advocate
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, Pennsylvania 17120

Frank Casolito, Action Chief
Bureau of Radiation Protection
Department of Environmental Protection
380 Scotch Road
Trenton, New Jersey 08628

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 28, 1983 and supplemented November 21, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

8506200416 850530
PDR ADOCK 05000311
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical Specifications

Date of Issuance: May 30, 1985

ATTACHMENT TO LICENSE AMENDMENT NO.37

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

| Remove Page | Insert Page |
|---|---|
| 3/4 7-17 | 3/4 7-17 |

SURVEILLANCE REQUIREMENTS (Continued)

d. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 4 inches Water Gauge while operating the system at a flow rate of 7410 cfm ± 10%.

2. Verifying that on a safety injection test signal or control room area high radiation test signal, the system automatically actuates in the recirculation mode by closing off the outside air supply and diverting air flow through the HEPA filter and charcoal adsorber bank.

3. Verifying that the system maintains the control room at a positive pressure of greater than or equal to 1/8 inch W.G. relative to the outside atmosphere during system operation with makeup air being supplied through the HEPA filters and charcoal adsorbers at the design makeup flow rate of less than or equal to 326 cfm.

e. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove greater than or equal to 99% of the DOP when they are tested in-place while operating the system at a flow rate of 7410 cfm ± 10%.

f. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove greater than or equal to 99% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place while operating the system at a flow rate of 7410 cfm ± 10%.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATION STATION, UNIT NO. 2

DOCKET NO. 50-311

## Introduction

By submittals dated September 28, 1983 and supplemented November 21, 1984, Public Service Electric and Gas Company requested a change in the surveillance requirements in the Technical Specifications for the control room emergency air conditioning system for Salem Unit 2. The request involved reducing the positive pressure at which the control room is maintained during leak test from 1/4 inch water gauge (W.G.) to 1/8 inch W.G. relative to the outside atmosphere.

## Evaluation and Summary

The value of 1/4 W.G. exceeds that which is currently required in the Standard Technical Specifications for control rooms (1/8 inch W.G.). Therefore, the staff finds this request acceptable. However, review of the licensee's request indicted that an important Standard Technical Specification parameter -- namely, the pressurization flow rate -- had been omitted from the surveillance requirement. Analyses of radiation dose to control room operators following a LOCA performed in the SER and the licensee's response to NUREG-0737, Item III.D.3.4, assume an unfiltered infiltration rate corresponding to a pressurization flow rate of 200 cubic feet per minute (CFM) at a relative pressure of 1/8 inch W.G. The staff would use this value in the specification. However, the licensee indicated that the pressurization system is designed to deliver a maximum of 300 cfm of filtered air into the control room following an accident. In addition, in discussions with the licensee, the licensee stated that the control room would be isolated upon detection of a high radiation signal and that pressurization would be result of operator actions, not an automatic function. The licensee's logic was that if the outside air was contaminated, then bringing this air into the control room could result in unnecessary doses to the operator.

The staff agreed with the licensee, but indicated that the licensee's logic would be acceptable if:

8506200417 850530
PDR ADOCK 05000311
P PDR

1. radiation detectors existed in both the control room and the outside air intakes which would allow the control room operators and health physics personnel to make a determination as to whether to remain in an isolated mode, or to pressurize the control room; and

2. the plant procedures should be revised to reflect consideration of the bases for determining when the control room should be pressurized.

In a letter dated November 21, 1984, the licensee verified the monitoring capability, and committed to revising the plant procedures for determining when to pressurize the control room. In a subsequent telephone conversation with the staff on February 21, 1985, licensee representatives indicated their intent to include considerations of the monitors and air handling equipment in the emergency procedure, and to provide health physics personnel with training related to design and operation of the air handling equipment and radiation monitors to ensure adequate post-accident dose management.

With respect to the test flow rate, the operators would be provided with better protection using a pressurization flow rate of 326 cfm (which is filtered) than allowing contaminated air leak into the control room in an isolated mode.

Therefore, the staff finds:

1. that testing of the control room at the 326 cfm designed flow rate is acceptable and the Technical Specifications should be modified accordingly; and

2. that the control room emergency response of isolation first, then on the basis of available monitoring, then determining whether pressurizing the control room would provide better operator protection under accident conditions is acceptable and meets the intent of GDC 19.

The licensee's submittal of November 21, 1984 was made to verify that procedures were in place, and did not contain substantive changes to the original submittal dated September 28, 1983.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no

significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 30, 1985

Principal Contributors:

K. Dempsey
T. Quay