

February 22, 1996

Mr. Leon R. Eliason
Chief Nuclear Officer - President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M93800 AND M93801)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment Nos. 180 and 161 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 28, 1995.

These amendments will remove "Reactor Coolant System - Chemistry" Specification 3/4.4.7 for Salem Unit 1 and Specification 3/4.4.8 for Salem Unit 2 and relocate these sections to the Salem Updated Final Safety Analysis Report, and the Surveillance Requirements and Limiting Conditions for Operation to applicable plant procedures controlled by the 10 CFR 50.59 process. The applicability will be changed from "At all times" to "Modes 1, 2, 3, 4, 5 and 6."

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000272
P PDR

Docket Nos. 50-272/311

- Enclosures: 1. Amendment No. 180 to License No. DPR-70
- 2. Amendment No. 161 to License No. DPR-75
- 3. Safety Evaluation

cc w/encls: See next page

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DATE	1/30/96	1/30/96	2/12/96	2/20/96	2/14/96

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 22, 1996

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
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Post Office Box 236
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SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M93800
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These amendments will remove "Reactor Coolant System - Chemistry" Specification 3/4.4.7 for Salem Unit 1 and Specification 3/4.4.8 for Salem Unit 2 and relocate these sections to the Salem Updated Final Safety Analysis Report, and the Surveillance Requirements and Limiting Conditions for Operation to applicable plant procedures controlled by the 10 CFR 50.59 process. The applicability will be changed from "At all times" to "Modes 1, 2, 3, 4, 5 and 6."

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Leonard N. Olshan".

Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-272/311

Enclosures: 1. Amendment No. 180 to
License No. DPR-70
2. Amendment No. 161 to
License No. DPR-75
3. Safety Evaluation

cc w/encls: See next page

Mr. Leon R. Eliason
Public Service Electric & Gas
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Salem Nuclear Generating Station,
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 180
License No. DPR-70

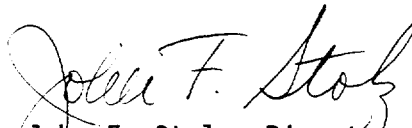
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 28, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 180, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 22, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 180

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

V
XII
3/4 4-17
3/4 4-18
3/4 4-19
B 3/4 4-4

Insert Pages

V
XII
3/4 4-17 through 3/4 4-19
-
-
B 3/4 4-4

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION	
Startup and Power Operation3/4 4-1
Hot Standby3/4 4-2
Hot Shutdown3/4 4-3
Cold Shutdown3/4 4-3b
3/4.4.2.1 SAFETY VALVES - SHUTDOWN3/4 4-4
3/4.4.2.2 SAFETY VALVES - OPERATING3/4 4-4a
3/4.4.3 RELIEF VALVES3/4 4-5
3/4.4.4 PRESSURIZER3/4 4-6
3/4.4.5 STEAM GENERATORS3/4 4-7
3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE	
Leakage Detection System3/4 4-14
Operational Leakage3/4 4-15
Pressure Isolation Valves3/4 4-16a
3/4.4.7 DELETED	
3/4.4.8 SPECIFIC ACTIVITY3/4 4-20
3/4.4.9 PRESSURE/TEMPERATURE LIMITS	
Reactor Coolant System3/4 4-24
Pressurizer3/4 4-29
Overpressure Protection Systems3/4 4-30
3/4.4.10 STRUCTURAL INTEGRITY	
ASME Code Class 1, 2, and 3 Components3/4 4-32
3/4.4.11 INTENTIONALLY BLANK3/4 4-34
3/4.4.12 HEAD VENTS3/4 4-35

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 PROTECTIVE INSTRUMENTATIONB 3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURES (ESF) INSTRUMENTATIONB 3/4 3-1
3/4.3.3 MONITORING INSTRUMENTATIONB 3/4 3-1
3/4.3.4 TURBINE OVERSPEED PROTECTIONB 3/4 3-4
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATIONB 3/4 4-1
3/4.4.2 SAFETY VALVESB 3/4 4-1a
3/4.4.3 RELIEF VALVESB 3/4 4-1a
3/4.4.4 PRESSURIZERB 3/4 4-2
3/4.4.5 STEAM GENERATORSB 3/4 4-2
3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGEB 3/4 4-3
3/4.4.7 DELETED	
3/4.4.8 SPECIFIC ACTIVITYB 3/4 4-5
3/4.4.9 PRESSURE/TEMPERATURE LIMITSB 3/4 4-6
3/4.4.10 STRUCTURAL INTEGRITYB 3/4 4-17
3/4.4.11 BLANKB 3/4 4-17
3/4.4.12 REACTOR VESSEL HEAD VENTSB 3/4 4-17

PAGES 3/4 4-17 THROUGH 3/4 4-19 DELETED

REACTOR COOLANT SYSTEM

BASES

3/4.4.6.2 OPERATIONAL LEAKAGE (Continued)

The total steam generator tube leakage limit of 1 GPM for all steam generators (but not more than 500 gpd for any steam generator) ensures that the dosage contribution from the tube leakage will be limited to a small fraction of Part 100 limits in the event of either a steam generator tube rupture or steam line break. The 1 GPM limit is consistent with the assumptions used in the analysis of these accidents. The 500 gpd leakage limit per steam generator ensures that steam generator tube integrity is maintained in the event of a main steam line rupture or under LOCA conditions.

PRESSURE BOUNDARY LEAKAGE of any magnitude is unacceptable since it may be indicative of an impending gross failure of the pressure boundary. Therefore, the presence of any PRESSURE BOUNDARY LEAKAGE requires the unit to be promptly placed in COLD SHUTDOWN.

3/4.4.7

THIS SECTION DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 28, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 161, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 22, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 161

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

V
XII
3/4 4-20
3/4 4-21
3/4 4-22
B 3/4 4-5

Insert Pages

V
XII
3/4 4-20 through 3/4 4-22
-
-
B 3/4 4-5

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION	
Startup and Power Operation	3/4 4-1
Hot Standby	3/4 4-2
Hot Shutdown	3/4 4-3
Cold Shutdown	3/4 4-4a
3/4.4.2 SAFETY VALVES - SHUTDOWN	3/4 4-5
3/4.4.3 SAFETY VALVES - OPERATING	3/4 4-6
3/4.4.4 PRESSURIZER	3/4 4-7
3/4.4.5 RELIEF VALVES	3/4 4-8
3/4.4.6 STEAM GENERATORS	3/4 4-9
3/4.4.7 REACTOR COOLANT SYSTEM LEAKAGE	
Leakage Detection System	3/4 4-16
Operational Leakage	3/4 4-17
3/4.4.8 DELETED	
3/4.4.9 SPECIFIC ACTIVITY	3/4 4-23
3/4.4.10 PRESSURE/TEMPERATURE LIMITS	
Reactor Coolant System	3/4 4-27
Pressurizer	3/4 4-30
Overpressure Protection Systems	3/4 4-31
3/4.4.11 STRUCTURAL INTEGRITY	
ASME Code Class 1, 2, and 3 Components	3/4 4-33
3/4.4.12 HEAD VENTS	3/4 4-34

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 PROTECTIVE INSTRUMENTATION	B 3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURES (ESF) INSTRUMENTATION	B 3/4 3-1
3/4.3.3 MONITORING INSTRUMENTATION	B 3/4 3-1
<u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION	B 3/4 4-1
3/4.4.2 SAFETY VALVES	B 3/4 4-2
and 3/4.4.3	
3/4.4.4 PRESSURIZER	B 3/4 4-2
3/4.4.5 RELIEF VALVES	B 3/4 4-2
3/4.4.6 STEAM GENERATORS	B 3/4 4-3
3/4.4.7 REACTOR COOLANT SYSTEM LEAKAGE	B 3/4 4-4
3/4.4.8 DELETED	
3/4.4.9 SPECIFIC ACTIVITY	B 3/4 4-6
3/4.4.10 PRESSURE/TEMPERATURE LIMITS	B 3/4 4-7
3/4.4.11 STRUCTURAL INTEGRITY	B 3/4 4-17

PAGES 3/4 4-20 THROUGH 3/4 4-22 DELETED

REACTOR COOLANT SYSTEM

BASES

3/4.4.7.2 OPERATIONAL LEAKAGE (Continued)

The total steam generator tube leakage limit of 1 GPM for all steam generators (but not more than 500 gpd for any steam generator) ensures that the dosage contribution from the tube leakage will be limited to a small fraction of Part 100 limits in the event of either a steam generator tube rupture or steam line break. The 1 GPM limit is consistent with the assumptions used in the analysis of these accidents. The 500 gpd leakage limit per steam generator ensures that steam generator tube integrity is maintained in the event of a main steam line rupture or under LOCA conditions.

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3/4.4.8

THIS SECTION DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 180 AND 161 TO FACILITY OPERATING
LICENSE NOS. DPR-70 AND DPR-75
PUBLIC SERVICE ELECTRIC & GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY
SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated September 28, 1995, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Salem Nuclear Generating Station, Unit Nos. 1 and 2, Technical Specifications (TSs). The requested changes will remove "Reactor Coolant System - Chemistry" Specification 3/4.4.7 for Salem Unit 1 and Specification 3/4.4.8 for Salem Unit 2 and relocate these sections to the Salem Updated Final Safety Analysis Report (UFSAR), and the Surveillance Requirements and Limiting Conditions for Operations to applicable plant procedures controlled by the 10 CFR 50.59 process. In addition, the applicability will be changed from "At all times" to "Modes 1, 2, 3, 4, 5 and 6."

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to state the TSs to be included as part of the license. The Commission's regulatory requirements related to the content of the TSs are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

The Commission has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the final policy statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled

documents, consistent with the standard enunciated in *Portland General Electric Co.* (Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TSs, as follows:

- (1) installed instrumentation that is used to detect, and indicate in the control room a significant abnormal degradation of the reactor coolant pressure boundary;
- (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;
- (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; and
- (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.¹

As a result, existing TS requirements which fall within or satisfy any of the criteria in the Final Policy Statement must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other licensee-controlled documents.

3.0 EVALUATION

The licensee has proposed relocating TS 3/4.4.7, "Reactor Coolant System - Chemistry," to the UFSAR. The reactor coolant chemistry program provides limits on particular chemical properties of the primary coolant, and

¹ The Commission recently adopted amendments to 10 CFR 50.36, pursuant to which the rule was revised to codify and incorporate these criteria. See Final Rule, "Technical Specifications," 60 FR 36953 (July 19, 1995). The Commission indicated that reactor core isolation cooling, isolation condenser, residual heat removal, standby liquid control, and recirculation pump trip systems are included in the TS under Criterion 4, although it recognized that other structures, systems, and components could also meet this criterion. (60 FR 36956)

surveillance practices to monitor those properties, to ensure that degradation of the reactor coolant pressure boundary is not exacerbated by poor chemistry conditions. However, degradation of the reactor coolant pressure boundary is a long-term process, and there are other, direct means to monitor and correct the degradation of the reactor coolant pressure boundary which are controlled by regulations and TSs; for example, in-service inspection and primary coolant leakage limits. Therefore, requirements related to the chemistry program do not constitute initial conditions that are assumed in any design basis accident or transient related to the Reactor Coolant System (RCS) integrity, nor does the reactor coolant chemistry program constitute a primary success path or risk-significant safety function warranting TS requirements under the criteria in the Final Policy Statement.

The licensee states that the reactor coolant chemistry requirements will be maintained in the UFSAR. Any changes to these chemistry requirements would be evaluated in accordance with 10 CFR 50.59 and if any changes are determined to involve an unreviewed safety question, the license must submit a license amendment to obtain prior NRC review and approval.

In addition, the licensee has proposed changing TS 3.4.7 applicability from "At all times" to "Modes 1,2,3,4,5 and 6." Currently, the licensee is required by the TSs to perform water chemistry sampling every 72 hours in all modes for the following parameters: dissolved oxygen, chloride, and fluoride. The proposed change will only require chemistry sampling during Modes 1, 2, 3, 4, 5 and 6.

For the Salem Units, moving all fuel to the fuel pool is a common outage activity and once the vessel is defueled this condition is commonly referred to as the "Mode Undefined." The Mode Undefined allows the licensee to take relief from certain equipment operability requirements in the TSs and allows them to drain water from the reactor vessel to support outage related work.

The Bases for the Technical Specification is to minimize and reduce the potential for RCS leakage or failure due to stress corrosion cracking. The effects of exceeding the oxygen, chloride and fluoride TS limits are time and temperature dependent. However, high stress and high temperature conditions that lead to stress corrosion, are absent in the Mode Undefined.

Also, with the relocation of the RCS - Chemistry from the TS to the UFSAR, plant procedure requirements will continue to provide adequate assurance that concentrations in excess of the limits will be detected.

In conclusion, the above relocated requirements relating to RCS - Chemistry are not required to be in the TS under 10 CFR 50.36 or §182a of the Atomic Energy Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. Further, they do not fall within any of the four criteria set forth in the Commission's Final Policy Statement, discussed above. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.59 [or

such other specific regulatory control as may be applicable in the particular instance] to assure continued protection of the public health and safety. Accordingly, the staff has concluded that these requirements may be relocated from the TS to the licensee's UFSAR.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 56369). The amendment also relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Zimmerman

Date: February 22, 1996