



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 9, 1989

Docket No. 50-311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: EMERGENCY TECHNICAL SPECIFICATION, TABLE 4.3-1
(TAC NO. 72987)

RE: SALEM GENERATING STATION, UNIT NO. 2

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. DPR-75 for the Salem Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 21, 1989.

This amendment has been prepared and issued on an emergency basis to permit Salem, Unit 2 to continue to operate at 100% rated thermal power. A discrepancy was found to exist between the distribution copy of the Salem, Unit 2, Technical Specifications and the NRC record copy of the Salem, Unit 2, Technical Specifications. If the requirements of the NRC record copy were applied, a required surveillance would not have been performed in the required time. This would make the Reactor Coolant Pump (RCP) Breaker Position Trip units inoperable. With all 4 RCP breaker position units inoperable, the plant is in a one hour shutdown action statement. The required surveillance requirements cannot be performed with the plant at power. A temporary waiver of compliance was granted on April 21, 1989, that permitted Public Service Electric and Gas to operate Unit 2 at up to 100% rated thermal power without having tested the Pump Trip during the last startup.

8905180303 890509
PDR ADOCK 05000311
F FDC

Handwritten signature and initials
JFO 1/1

A copy of our safety evaluation is also enclosed. Notice of Issuance of amendment to Facility Operating License and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Bruce A. Boger".

Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 73 to
License No. DPR-75
2. Safety Evaluation

cc w/enclosures:
See next page

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Sincerely,

/S/

Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

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2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION w/enclosures:

Docket File
MO'Brien (2)
Wanda Jones
Brent Clayton
NRC PDR
OGC
EButcher
EWenzinger
Local PDR
DHagan
Tech Branch
PDI-2 Reading
EJordan
ACRS (10)
WButler
BGrimes
CMiles, GPA/PA
JStone/MThadani
TMeek (4)
RDiggs, ARM/LFMB

[SALEM AMEND]

PDI-2/PA
MO'Brien
5/2/89

PDI-2/PM
UDFischer:tr
5/2/89

OGC
5/12/89

PDI-2/D
WButler
5/2/89

DRP/ADRI
BBoger
5/19/89

Mr. Steven E. Miltenberger
Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

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Vice President - Nuclear Operations
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Hancocks Bridge, New Jersey 08038

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Lower Alloways Creek Township
Municipal Hall
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Assistant Consumer Advocate
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Department of Law and Public Safety
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Trenton, NJ 08625

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c/o Jack Urban
General Manager, Fuel Supply
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Wilmington, DE 19899

Mr. David M. Scott, Chief
Bureau of Nuclear Engineering
Department of Environmental Protection
State of New Jersey
CN 411
Trenton, NJ 08625



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated April 21, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

8905180305 890509
PDR ADOCK 05000311
P PDC

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of April 21, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 9, 1989

PDI-2/DA
WButler
5/2/89

PDI-2/PM
JStone
5/2/89

PDI-2/D
WButler
5/13/89

PDI-2/D
WButler
5/12/89

5/9

(2) Technical Specifications and Environmental Protection Plan

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FOR THE NUCLEAR REGULATORY COMMISSION



Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 9, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

3/4 3-12

Insert Page

3/4 3-12

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| <u>FUNCTIONAL UNIT</u> | <u>CHANNEL CHECK</u> | <u>CHANNEL CALIBRATION</u> | <u>CHANNEL FUNCTIONAL TEST</u> | <u>MODES IN WHICH SURVEILLANCE REQUIRED</u> |
|--|--------------------------|--------------------------------|--|---|
| 13. Loss of Flow - Two Loops | S | R | N.A. | 1 |
| 14. Steam Generator Water Level-- Low-Low | S | R | M | 1, 2 |
| 15. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level | S | R | M | 1, 2 |
| 16. Undervoltage - Reactor Coolant Pumps | N.A. | R | M | 1 |
| 17. Underfrequency - Reactor Coolant Pumps | N.A. | R | M | 1 |
| 18. Turbine Trip | | | | |
| a. Low Autostop Oil Pressure | N.A. | N.A. | S/U(1) | N.A. |
| b. Turbine Stop Valve Closure | N.A. | N.A. | S/U(1) | N.A. |
| 19. Safety Injection Input from SSPS | N.A. | N.A. | M(4) | 1, 2 |
| 20. Reactor Coolant Pump Breaker Position Trip | N.A. | N.A. | R | N.A. |
| 21. Reactor Trip Breaker | N.A. | N.A. | S/U(10), M(11) and SA(12) | 1, 2 and * |
| 22. Automatic Trip Logic | N.A. | N.A. | M(5) | 1, 2 and * |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DRP-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NO. 2

DOCKET NO. 50-311

1.0 INTRODUCTION

By letter dated April 21, 1989, Public Service Electric & Gas Company (PSE&G) requested an amendment to Facility Operating License No. DPR-75 for the Salem Generating Station, Unit No. 2. The proposed amendment would revise the Unit 2 Technical Specification, Table 4.3-1 Functional Unit 20 to agree with the Unit 1 Technical Specifications and the Westinghouse Standard Technical Specifications. This change is being submitted on an emergency basis because a discrepancy was found to exist between the distribution copy of the Salem, Unit 2, Technical Specifications and the NRC record copy of the Salem, Unit 2, Technical Specifications. If the requirements of the NRC record copy are applied, a required surveillance would not have been performed in the required time. This would make the Reactor Coolant Pump (RCP) Breaker Position Trip units inoperable. With all 4 RCP breaker position units inoperable, the plant is in a one hour shutdown action statement. The required surveillance requirements cannot be performed with the plant at power. On April 24, 1989, PSE&G submitted the affidavit along with a copy of the April 21, 1989 submittal. No technical changes were made in this resubmittal.

2.0 EVALUATION

In its April 21, 1989 letter, Public Service Electric and Gas Company submitted a proposed change to the Technical Specifications for the Reactor Coolant Pump Breaker Position Trip (Position Trip) for the Salem Generating Station, Unit 2. The proposed change would reduce the surveillance tests of the Reactor Coolant Pump Breaker Position Trip from once per startup to once per refueling outage. This would make the Unit 2 Technical Specification 4.3-1, Reactor Coolant Pump Breaker Position Trip consistent with the Unit 1 Technical Specification and be consistent with accepted industry practices for surveillance of this trip. The change

from once per startup to once per refueling outage reduces the number of unnecessary equipment challenges and therefore, increases the probability that the equipment will function as intended. The reactor trip associated with the Reactor Coolant Pump Breaker Position Trip is not taken credit for the Salem accident analysis and does not reduce the margin of safety.

We have reviewed the licensee's evaluation and justification and find it acceptable. As such, we find that the proposed Technical Specification change is acceptable. This position is consistent with the related surveillance test interval specified in other PWR technical specifications.

3.0 EMERGENCY BASIS

In its April 21, 1989 letter, the licensee has shown that Unit 2 is currently operating at 100% power and would be required to enter a one hour shutdown action statement if the requirements of the NRC record copy of the Unit 2 Technical Specifications were applied. The distribution copies of the Salem, Unit 2, Technical Specifications do not contain this action statement, hence, the shutdown would not be required.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission has provided standards for determining whether a significant hazards consideration exists (10 CFR 50.92(c)). A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The staff has reviewed the licensee's request and concurs with the following basis and conclusions provided by the licensee in its April 21, 1989 submittal.

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change is consistent with accepted industry practices for surveillance of the Reactor Coolant Pump Breaker Position Trip equipment. The change from once per startup to once per refueling reduces the number of unnecessary equipment challenges and, therefore, increases the probability that the equipment will function as intended. The reactor trip associated with the reactor coolant pump breaker position is not taken credit for in the Salem accident analysis.

- II. The proposed change does not create the possibility of a new or different kind of accident.

Since the proposed change does not change any plant hardware nor the manner in which the equipment functions, a new or different kind of accident is not created.

- III. The proposed change does not reduce the margin of safety.

Because the proposed change makes Salem, Unit 2 consistent with accepted industry practice and since the reactor trip associated with the reactor coolant pump breaker position is not taken credit for in the Salem accident analysis, the margin of safety is not reduced.

5.0 STATE CONSIDERATION

The State of New Jersey was contacted on May 1, 1989 and they had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) the amendment does not (a) significantly increase the probability or consequences of an accident previously evaluated, (b) increase the possibility of a new or different kind of accident from any previously evaluated or (c) significantly reduce a safety margin and, therefore, the amendment does not involve significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributors: B. Marcus and D. Fischer

Dated: May 9, 1989